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## Radiological Protection from Naturally Occurring Radioactive Material (NORM) in Industrial Processes

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40 **Abstract**– The purpose of this Publication is to provide guidance on radiological protection  
41 in industries involving NORM. Industries involving NORM give rise to multiple hazards and  
42 the radiological hazard is not necessarily dominant. Such industries are diverse and may  
43 involve exposure to people and the environment where protective actions need to be  
44 considered. NORM presents no real prospect of a radiological emergency leading to tissue  
45 reactions or immediate danger for life. However, the accidental release of large volumes of  
46 NORM may result in detrimental effects on the environment, including of radiological  
47 nature. NORM associated with industrial processes is an existing exposure situation, except  
48 when NORM is used for its radioactive properties which should be addressed on the basis of  
49 the principles of justification (of the actions taken) and optimisation of the protection above  
50 or below appropriate reference levels. Radon and thoron exposures should be managed as  
51 recommended in *Publication 126*.

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53 An integrated approach to NORM processes is recommended, starting with characterisation  
54 of the situation and protection strategies already implemented to manage other workplace  
55 hazards, and then assessing the need for additional actions. The selection and implementation  
56 of protection strategies for workers should be a graded response to the magnitude of the  
57 hazards. According to the characteristics of the exposure situation, notably the actual and  
58 potential exposure pathways, the individual dose distribution and the prospect for  
59 optimisation, an appropriate reference level can be selected, either below a few mSv per year  
60 or above a few mSv if necessary, but very rarely exceeding 10 mSv per year. In the same  
61 line, control of the workplace and the conditions of work are used to reduce the risk, while  
62 the control of workers enters when adequate protection has not already been achieved with  
63 workplace controls.

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65 A graded approach should be used in implementing requirements. Public exposure should be  
66 dealt with through the control of discharge, waste and residue, after characterisation of the  
67 situation. The reference level for the protection of the public should be selected below a few  
68 mSv per year. The protection of non-human species should be dealt with as part of an  
69 environmental assessment, taking into account all hazards and impacts. This should include  
70 identification of exposed organisms in the environment and using relevant derived  
71 consideration reference levels (DCRL), to ascertain the magnitude of the impacts and inform  
72 decisions on options for control of exposure.

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## EDITORIAL

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104 *To be drafted*

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**MAIN POINTS**

- 112 • **Situations involving Naturally Occurring Radioactive Material (NORM) are**  
113 **existing exposure situations except when NORM is used for its radioactive**  
114 **properties.**
- 115 • **NORM industrial activities are controllable, and protection is achieved through**  
116 **optimisation using reference levels.**
- 117 • **Protective actions may need to be considered with regard to external exposure,**  
118 **intake of radioactive material, and radon or thoron inhalation. Radon and thoron**  
119 **exposures should be managed as recommended in *Publication 126*.**
- 120 • **NORM presents no real prospect of a radiological emergency leading to tissue**  
121 **reactions or immediate danger to life, but may pose an issue of environmental**  
122 **contamination.**
- 123 • **An integrated and graded approach to protection is recommended, starting with**  
124 **strategies already implemented to manage other workplace hazards.**  
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## 1. INTRODUCTION

### 1.1. Background

128 (1) All minerals and raw materials of a geological nature may contain radionuclides of  
129 natural origin. The main radionuclides of interest are  $^{40}\text{K}$  and radionuclides from the  $^{232}\text{Th}$   
130 and  $^{238}\text{U}$  decay series. Thorium-232 and  $^{238}\text{U}$  decay through a series of radionuclides to stable  
131 isotopes  $^{208}\text{Pb}$  and  $^{206}\text{Pb}$ , respectively. These decaying radionuclides are known as daughter  
132 radionuclides or progeny. The other primordial radionuclides are of much lower abundance.

133 (2) For most human activities involving minerals and raw materials, the level of  
134 exposure due to primordial radionuclides decay series is not a concern for radiological  
135 protection. However, there are a number of circumstances in which materials containing  
136 natural radionuclides are recovered, processed, used, or moved such that enhanced radiation  
137 exposures may result. Material involved in processes giving rise to these enhanced exposures  
138 is considered to be Naturally Occurring Radioactive Material (NORM). For example, certain  
139 minerals (e.g. zirconium, monazite), including some that are commercially exploited, may  
140 contain potassium and/or thorium and/or uranium progeny at significant concentrations.

141 (3) Furthermore, during the extraction of minerals and their processing, the  
142 radionuclides may become unevenly distributed between the products, by-products,  
143 discharge, residue or waste arising from the process(es). The radionuclide activity  
144 concentrations may exceed those in the original mineral, sometimes by several orders of  
145 magnitude, which in turn can significantly increase the exposure of workers and/or members  
146 of the public as well as lead to the contamination of the environment.

147 (4) Only a few years after the discovery of radioactivity by A. H. Becquerel in 1896,  
148 radon – or “radium emanation” as it was called, was found in petroleum and in natural gas  
149 brought to the surface. In 1898 Marie Curie identified radium and polonium after processing  
150 several tons of pitchblende, an ore with high uranium content. Later, several investigations  
151 led to the first general review of the radioactivity associated with sedimentary rocks,  
152 petroleum, underground water and brines (Monicard, 1952). The discovery of radioactive  
153 scales from natural sources in British and American oil production facilities was first  
154 mentioned in the 1950’s (Schmidt, 2000). However, the potential health, safety and  
155 environmental risks due to radiation exposure from NORM in the industry were only widely  
156 realised since the 1980’s (Miller et al., 1991).

157 (5) In *Publication 26* (ICRP, 1977), ICRP recognised that some practices may “increase  
158 the level of exposure from the natural background of radiation” (Para. 235) and that there  
159 may be levels of natural radiation that might have to be controlled in much the same way as  
160 for artificial sources. The Commission did not give practical guidance on the principles for  
161 such control. In the same year, UNSCEAR introduced for the first time a chapter on  
162 ‘technologically enhanced exposures to natural radiation’ in its report to the General  
163 Assembly (UNSCEAR, 1977).

164 (6) In *Publication 39* (ICRP, 1984) and later in *Publication 60* (ICRP, 1991), the  
165 Commission proposed principles for limiting exposures of workers and the public to natural  
166 sources of radiation and notably primordial radionuclides and progeny. The Commission  
167 stated that there should be requirements to include some exposures to natural sources as part  
168 of occupational exposures when it comes to ‘operations with and storage of materials not  
169 usually regarded as radioactive, but which contain significant traces of natural radionuclides’  
170 (ICRP, 1991 Para. 136).

171 (7) In *Publication 82* (ICRP, 1999) devoted to the protection of the public against  
172 prolonged exposures, the Commission first acknowledged the term ‘NORM’ by noting:  
173 “industrial development has further increased the ‘natural’ exposure of people by  
174 technologically enhancing the concentrations of radionuclides in naturally occurring  
175 radioactive materials (NORMs)” (Para. 6). The Publication then focused on the application of  
176 the system described in *Publication 60* for radiological protection to practices resulting in  
177 prolonged exposure. Optimisation was expected to be applied to ensure that doses were ‘as  
178 low as reasonably achievable’ taking into account economic and social factors. The  
179 Commission later provided detailed guidance on the application of the optimisation principle  
180 in ‘The Optimisation of Radiological Protection: Broadening the Process’ (ICRP, 2006, Part  
181 2). This publication recommended that dose constraints and dose limits for practices may be  
182 appropriate to NORM exposure, but should be applied with ‘care and flexibility’.

183 (8) In *Publication 103* (ICRP, 2007a), the Commission revised the system for  
184 radiological protection of *Publication 60*. The approach is now based on the characteristics of  
185 the radiation exposure situation rather than the process-based approach previously employed.  
186 The system applies to all exposures to ionising radiation, from any source, regardless of size  
187 or origin, but apply in their entirety only to situations in which either the source of the  
188 exposure or the pathways leading to doses received by individuals can be controlled by some  
189 reasonable means.

190 (9) A major implication of this is that all exposures, including those from naturally  
191 occurring radiation sources, are now within the scope of the system and that the principles of  
192 justification and optimisation always apply. Exposures from natural sources are considered to  
193 be existing exposure situations.

194 (10) *Publication 104* (ICRP, 2007b) recognised that there is a need for international  
195 consensus on NORM exposure management and that industries involving NORM have been  
196 regulated variably with regard to radiological protection, because the radiological protection  
197 system has been introduced after the start of operation, and existing industrial hygiene  
198 controls already limit the potential for radiation exposure (e.g. control of airborne dust).  
199 Exclusion and exemption of industries involving NORM and activities using numerical  
200 criteria may be useful but lack the quantitative judgement that is also often necessary. Hence,  
201 *Publication 104* advocated a graded approach in the management of NORM exposure, taking  
202 into account the prevailing circumstances and the risk to people, with the global aim of  
203 promoting the protection of workers and public health (Para. 137).

204 (11) The Commission has recently engaged in a set of Publications dedicated to applying  
205 the system of radiological protection to existing exposure situations. *Publication 126* (ICRP,  
206 2014b) updated the recommendations for the protection against exposure to radon.  
207 *Publication 132* (ICRP, 2016) is devoted to Radiological Protection from Cosmic Radiation  
208 in Aviation. *Publications 109* and *111* on Emergency Exposure Situations and Living in  
209 Long-term Contaminated Areas following a Radiological Emergency are currently being  
210 updated. A Publication is also in preparation dedicated to exposures resulting from  
211 contaminated sites from past industrial, military and nuclear activities.

## 212 1.2. Scope

213 (12) This publication elaborates on management of existing exposure situations with  
214 regard to NORM. The Commission’s approach to NORM builds on *Publication 103* (the 2007  
215 Recommendations), *Publication 124* (environment) and *Publication 126* (radon and thoron).  
216 For the purpose of management of NORM as an existing exposure situation, previous advice

217 may be considered superseded. The focus is upon industrial processes such as mining and  
218 mineral extraction, or other industrial activities that may lead to exposures to NORM of  
219 geological origin, which have been identified as requiring consideration of radiological  
220 protection. The term ‘industrial’ also includes small-size business activities. In many cases,  
221 the input to the process does not have elevated levels of NORM (e.g. fossil fuels); however,  
222 the subsequent industrial processes generate higher concentration of radionuclides in the  
223 products, by-products, discharge, residue or waste. The industrial processes may also increase  
224 the exposure of workers and/or members of the public and/or lead to discharges of  
225 radioactive substances to the environment. More details about activities that may involve  
226 NORM exposure are given in chapter 2 and Appendix 1.

227 (13) Some mining facilities, however, have been established for the expressed purpose of  
228 extracting materials such as uranium and thorium from ore to be used for their radioactive,  
229 fissile or fertile properties. These industries are considered as planned exposure situations  
230 under the current system of radiological protection as outlined by the Commission in ICRP,  
231 2007 and are not the subject of this publication.

232 (14) One contributor to NORM exposures is usually radon ( $^{222}\text{Rn}$ ) gas (from the decay of  
233  $^{238}\text{U}$ ) and, to a lesser extent, thoron ( $^{220}\text{Rn}$ ) gas (from the decay of  $^{232}\text{Th}$ ). ICRP recently  
234 provided information on lung cancer risk from radon and thoron by reviewing  
235 epidemiological studies (ICRP, 2010), formulated recommendations for the protection of  
236 workers and public against them (ICRP, 2014b) and provided new dose coefficients for radon  
237 (ICRP 2017). In *Publication 126* (ICRP, 2014b), the Commission recommends an integrated  
238 approach for controlling radon exposure, relying as far as possible on the management of  
239 buildings or locations in which radon exposure occurs, whatever the use of the building. This  
240 approach is valid for radon and thoron arising from different sources in the workplace (e.g.  
241 from the ground, building materials and from minerals containing NORM). Thus, radon and  
242 thoron exposures in industries involving NORM should be managed in accordance with the  
243 approach of *Publication 126* and will not be considered explicitly in this publication.

244 (15) Due to the long-standing history of many industries involving NORM, sites have  
245 been identified as contaminated by NORM residues and wastes from past activities (legacy  
246 sites). In 2014, ICRP established a Task Group to develop a report on how to apply the  
247 Commission’s recommendations to exposures resulting from contaminated sites from past  
248 industrial activities, so this topic will not be fully addressed here.

249 (16) The 2007 Recommendations (ICRP, 2007a) extended the system of radiological  
250 protection to address protection of the environment, including flora and fauna, more  
251 explicitly. Later, in *Publication 124* (ICRP, 2014a), the Commission describes its framework  
252 for protection of the environment, through the introduction of Reference Animals and Plants  
253 and how it should be applied within the system of radiological protection. Consistent with the  
254 approach established by the 2007 Recommendations (ICRP, 2007a), this Publication will  
255 specifically address the protection of the environment against NORM exposure.

256 (17) The ethical underpinnings of the system of radiological protection rely on four core  
257 ethical values as described in the *Publication 138* (ICRP, 2018): beneficence/non-  
258 maleficence, prudence, justice and dignity. There are important ethical issues to be integrated  
259 in the protection strategy against NORM exposure. Applying the system of protection is a  
260 permanent quest for decisions that do more good than harm (beneficence/non-maleficence),  
261 that avoid unnecessary risk (prudence), that establish a fair distribution of exposures (justice)  
262 and treat people with respect (dignity).

263 (18) While ionising radiation may be a consideration in terms of the protection of people  
264 and the environment from NORM, it is generally neither the only hazard nor the most  
265 dominant hazard. Indeed, many NORM residue and waste may contain toxic non-radiological

266 constituents that may be harmful to human health and/or the environment (e.g. heavy metals).  
267 The present Publication will not provide guidance on the management of these constituents,  
268 which may have to be controlled by environmental regulation. However, the Commission  
269 recommends the use of an integrated approach for the management of radiation and other  
270 hazards.

### 271 **1.3. Structure of this publication**

272 (19) Chapter 2 presents the characteristics of NORM exposures, an overview of the  
273 industries and practices where NORM exposure can occur, and elements related to the  
274 NORM cycle. Chapter 3 describes the Commission's system of radiological protection  
275 applied to NORM exposure, including the type of exposure situation, the category of  
276 exposure concerned and the basic principles to be applied. Chapter 4 provides guidance on  
277 the implementation of the system of radiological protection using an integrated and graded  
278 approach for the various exposed workers, public and the environment. Conclusions are  
279 provided in chapter 5. Appendix 1 completes Chapter 2 with more details about activities that  
280 may involve NORM exposure.  
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## 2. CHARACTERISTICS OF EXPOSURE FROM NORM

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### 2.1. Ubiquity and variability

284 (20) Radionuclides from natural origin are ubiquitous and are present in almost all  
285 materials on Earth. They are in general not of radiological concern. Some human activities,  
286 however, have the potential to enhance radiation exposures from these materials.

287 (21) Many organisations have produced comprehensive reviews of industries that may  
288 cause NORM-related radiation exposure of workers, the public and the environment  
289 (UNSCEAR 1982, 2008; EC, 1999a; IAEA, 2006; EURATOM, 2013). Examples are given  
290 below. Further, previous industrial sites could have involved NORM, and these legacy sites  
291 may require attention. Details on these work activities are provided in Appendix 1.

- 292 1. Extraction of rare earth elements.
- 293 2. Production and use of metallic thorium and its compounds (i.e. for their metallic not  
294 fissile radioactive properties).
- 295 3. Mining and processing of ores (other than uranium or thorium for the nuclear fuel cycle).
- 296 4. Oil and gas recovery process.
- 297 5. Manufacture of titanium dioxide pigments.
- 298 6. The phosphate processing industry.
- 299 7. The zircon and zirconia industries.
- 300 8. Production of metal (tin, copper, iron, steel, aluminium, niobium/tantalum, bismuth,  
301 etc.).
- 302 9. Combustion of fossil fuel (mainly coal).
- 303 10. Water treatment.
- 304 11. Geothermal energy production.
- 305 12. Cement production, maintenance of clinker ovens.
- 306 13. Building materials (including building materials manufactured from residues or by  
307 products).

308 (22) Typical industries involving NORM process a wide range of raw materials with  
309 different levels of activity concentrations, producing a variety of products, by-products,  
310 discharges, residues and wastes. These industries may or may not be of radiological concern  
311 depending on the activity concentrations in the raw materials handled, the processes adopted,  
312 the uses of final products, the reuse and recycling of residues and the disposal of wastes.

313 (23) The range of process broadly leads to three scenarios for radiation exposure:

- 314 • From large quantities of material as an ore or a stockpile of raw material;
- 315 • From small quantities of material with concentrated radionuclides such as mineral  
316 concentrates, scales and sludge;
- 317 • From material that has been volatilised in high-temperature processes, like slags,  
318 precipitator dust and furnace fume.

319 (24) Work with NORM can give rise to external and internal radiation exposures.  
320 External exposures can arise from extended exposures to low (gamma) dose rates, from  
321 shorter exposures to high (gamma and sometimes beta) dose rates from performing  
322 maintenance on internals of equipment, slags, scales and sludges, or a combination of these.  
323 The potential for internal exposure is governed mostly by the way NORM occurs in the  
324 workplace, and the personal protective equipment worn by workers. Radon may be an  
325 important source of exposure in indoor or underground atmosphere (as mentioned above,  
326 radon exposure should be dealt with in accordance with *Publication 126* (ICRP, 2014b)). In  
327 large-scale mining and milling operations, airborne dust is a common industrial hazard, and

328 internal exposures from inhalation of NORM can be significant, especially where higher  
329 activity concentrations are present (e.g. above tens of Bq g<sup>-1</sup>). In contrast, internal exposures  
330 from ingestion of NORM, including in water, are usually low (EC, 1999a). However, there  
331 can be considerable differences depending on workplace conditions, the radionuclides  
332 involved and the physical and chemical matrices in which the radionuclides are incorporated  
333 (UNSCEAR, 2016).

334 (25) Very large numbers of workers in the world may be exposed to NORM, and  
335 although the data are more limited than those for occupational exposures to man-made  
336 sources, the worldwide level of exposure for workers exposed to natural sources of radiation  
337 has been estimated to 30,000 man.Sv annually (around 13 million workers) (UNSCEAR,  
338 2008). Until implementation of the International Basic Safety Standards for protection  
339 against ionising radiation in 1996 (IAEA, 1996) most countries had not been particularly  
340 concerned with assessing occupational exposure to natural sources of radiation. Table 2.1  
341 (adapted from IAEA (2006)) gives recent ranges of exposures to workers in some industries  
342 involving NORM. In the majority of workplaces, both the average and the maximum  
343 assessed doses received by workers are below a few mSv per year but higher doses – in some  
344 cases as high as few tens of mSv – may occur in certain situations and in specific workplaces  
345 (around 100 mSv y<sup>-1</sup> in very few underground mines).

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Table 2.1. Examples of dose assessments for workers (external and internal from dust, excluding exposure to radon)

Activities	Radionuclides with highest activity concentration	Annual effective dose (mSv y <sup>-1</sup> )			
		Minimum	Mean	Maximum	Distribution
Processing of thorium concentrate <sup>a</sup>	<sup>232</sup> Th (in feedstock and product)	3.0		7.8	
Production of thorium compounds <sup>b</sup>				82	67% <1
Mining of rare earth ore <sup>c</sup>	<sup>238</sup> U and <sup>232</sup> Th series (feedstock)		0.24 – 1		
Beneficiation of rare earth ore <sup>c</sup>			0.28 – 0.61		
Handling of monazite	<sup>232</sup> Th series			0.3	
Rare earth separation and purification	<sup>228</sup> Ra (residues)			0.3	
Decommissioning of a rare earths plant <sup>d</sup>	<sup>228</sup> Ra (residues)	0.2	7.2	8.94	
Mining of ore other than uranium ore	<sup>238</sup> U and <sup>232</sup> Th series (in general)	1.3	3	5	
Oil and gas production, offshore	<sup>226</sup> Ra (scale/sludge)			0.5	
Oil and gas production, onshore				0.05	
Oil production, cleaning of pipes <sup>c,e</sup>			0.6	3	80% <1
Titanium dioxide pigment production	• <sup>232</sup> Th (feedstock) • <sup>226</sup> Ra, <sup>228</sup> Ra (scale)			0.27	
Phosphate ore storage	<sup>238</sup> U series			0.28	
Phosphate fertiliser production	• <sup>238</sup> U (feedstock and product) • <sup>226</sup> Ra (residues)			0.5	
Zircon production	• <sup>238</sup> U series (feedstock)			0.4	
Bastnäsite (zirconia) production	• <sup>210</sup> Po (in dust precipitator)			0.4	
Manufacture and use of zircon ceramics	• <sup>238</sup> U (in fused zirconia/product)	0		2.3	87% <1
Manufacture of zircon/zirconia ceramics		~0.01		1.5	98% <1
Processing of Sn, Al, Ti and Nb ores	• <sup>232</sup> Th (feedstock, product and slag) • <sup>228</sup> Ra (residue)	0		3.2 <sup>f</sup>	69% <1
Copper smelting	<sup>226</sup> Ra (slag)			<1	
Recycling of metal scrap	<sup>210</sup> Po, <sup>210</sup> Pb (precipitator dust)			Negligible	
Coal mining	• <sup>238</sup> U • <sup>226</sup> Ra, <sup>228</sup> Ra (for coal with high Ra inflow water)		2.75		
Combustion of coal	<sup>210</sup> Po (scale)	0		0.4	
Combustion of coal				<1	
Combustion of coal				0.13	
Drinking water treatment	<sup>226</sup> Ra (sludge)			<1	
Manufacture of mineral insulation <sup>g</sup>	n.a	0.0011		0.0173	

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<sup>a</sup> Doses include contributions from inhalation of thoron.  
<sup>b</sup> Doses >1 mSv y<sup>-1</sup>, mainly due to dust inhalation, were identified in two of the six workplaces investigated. The assessment is being repeated after the implementation of dose reduction measures (equipping workers with respiratory protection, cleaning the workplaces periodically and installing air filtration).  
<sup>c</sup> Dose from external exposure only.  
<sup>d</sup> Doses received over a 9-month decommissioning period.  
<sup>e</sup> Doses received over a 5-month refurbishment period.  
<sup>f</sup> The maximum dose was 6 mSv prior to 2008.  
<sup>g</sup> The minerals were coal, bauxite, basalt and cement.

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361 (26) In terms of public exposure, direct external exposures (i.e. from NORM on the site)  
 362 are usually negligible, although there are exceptions to this. For some specific industry  
 363 involving NORM sites, it has been reported that some representative individuals in close  
 364 proximity to the plant can receive annual doses in the millisievert range (UNSCEAR, 2008).  
 365 In general, public doses from NORM mainly arise from radionuclides released to air and  
 366 water as routine discharges, and the use of NORM-containing by-products in commodities  
 367 such as building materials. A complete review is made difficult by the diversity of industries  
 368 involved, the local circumstances associated with the exposures, and the overall lack of site-  
 369 specific radiological assessments. Table 2.2 presents some data related to public exposures  
 370 from NORM (adapted from IAEA (2010)). These estimates are subject to uncertainties and  
 371 are often conservative. In Table 2.2 the annual effective dose from NORM to public is  
 372 estimated to be far below 1 mSv per year, except in the situation of wide use of  
 373 phosphogypsum in building material.

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375 Table 2.2. Examples of dose assessments for members of the public (excluding exposure to  
 376 radon).

Activities	Radionuclides with highest activity concentration	Annual effective dose (mSv)
Mining of rare earth ore	<sup>232</sup> Th (contaminated soil)	0.044
Beneficiation of rare earth ore	<sup>232</sup> Th (contaminated soil)	0.043
Use of slag from rare earths and steel production in house bricks	<sup>226</sup> Ra, <sup>232</sup> Th (bricks)	~0.2
Production of Th welding rods	N.A.	Negligible
Mining of ore other than uranium ore		Specified only as <1
Large mineral residue deposit, 1 Bq g <sup>-1</sup> <sup>238</sup> U and/or <sup>232</sup> Th	<sup>232</sup> Th and <sup>238</sup> U series	0.05–0.26
Oil and gas production	N.A.	Specified only as <1
Elemental phosphorus production		<0.04
Use of dicalcium phosphate animal feed	<sup>210</sup> Po, <sup>210</sup> Pb (in chicken)	<0.02
Use of phosphogypsum for agriculture	<sup>226</sup> Ra (in fertiliser)	Negligible
Use of phosphogypsum (PG) for construction of houses:	<sup>226</sup> Ra (in the building material)	
Walls and ceilings, PG panels,		0.02 – 0.2
Walls, ceilings and floor, hollow PG panels		0.46
Walls, ceilings and floor, solid PG panels		4.5
Walls, PG plasterboard lining		0.15 (India) or insignificant (Australia)
Walls, PG in bricks and cement		≤1.4
Manufacture of zircon/zirconia ceramics		Negligible
Steel production	<sup>232</sup> Th, <sup>228</sup> Ra (in dust/gaseous effluent)	<0.01
Use of metal recycling slag for road construction	<sup>223</sup> Ra (slag)	Specified only as <1
Combustion of coal	N.A.	Negligible
Drinking water treatment	N.A.	Negligible
Disposal of water treatment residue in landfill	<sup>226</sup> Ra (sludge)	0.01
Effluent water treatment, former U mine	N.A.	Specified only as <1
Use of common building materials for house construction	N.A.	<0.3 – 1

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379 (27) *Publication 103* (ICRP, 2007a) introduced an approach for developing a framework  
380 to demonstrate radiological protection of the environment. However, there are as yet few  
381 examples of the assessment of the impact of NORM, outside of uranium mining activities (or  
382 similar) on the environment. Each case should be evaluated on an individual basis taking all  
383 the present hazards, all concerned species, main environmental conditions and other  
384 characteristics into the consideration.

## 385 2.2. The NORM cycle

386 (28) Several stages of production involving NORM can be identified – some industries  
387 may involve almost all these stages, others may involve only some of them:

- 388 1. Mineral extraction,
- 389 2. Mineral processing,
- 390 3. Fabrication of products,
- 391 4. Use of products and by-products,
- 392 5. Re-use and recycling of residues,
- 393 6. Management of wastes, and
- 394 7. Dismantling or remediation and rehabilitation.

395 (29) The presence of NORM with elevated radionuclide concentrations could be an issue  
396 at any stage and may lead to significant radiological exposures of workers and the public and  
397 contamination of the environment if not adequately controlled.

398 (30) By-products and residues from a one industry involving NORM can be used as  
399 feedstock by other industry involving NORM and/or in common practices (e.g. building  
400 materials). In that sense, after being brought to surface (or else introduced into the industrial  
401 sector by another means), NORM enters a cycle, which is possibly endless (i.e. NORM can  
402 be moved and/or reprocessed from place to place), and enhanced exposures due to NORM  
403 may occur during all stages of the cycle.  
404

405

406

### 3. THE APPLICATION OF THE COMMISSION'S SYSTEM OF RADIOLOGICAL PROTECTION TO NORM

407

#### 3.1. Types of exposure situations and categories of exposure

##### 3.1.1. Types of exposure situations

410 (31) The Commission defines an exposure situation as a *network of events and situations*  
411 that begins with a natural or artificial radiation source, the transfer of the radiation or  
412 radioactive materials through various pathways, and the resulting exposure of individuals or  
413 the environment (ICRP, 2007a, Para. 169). Protection can be achieved by taking action at the  
414 source, or at any point in the exposure pathways of the exposed individuals.

415 (32) According to Para. 176 of *Publication 103* (ICRP, 2007a), the Commission intends  
416 its Recommendations to be applied to all sources in the following three types of exposure  
417 situations, which address all conceivable circumstances:

- 418 • *Existing exposure situations* are exposure situations resulting from a source that already  
419 exists, with no intention to use the source for its radioactive properties, before a decision  
420 to control the resulting exposure is taken. Decisions on the need to control the exposure  
421 may be necessary but not urgent. Characterisation of exposures is a prerequisite for their  
422 control;
- 423 • *Planned exposure situations* are situations resulting from the deliberate introduction and  
424 operation of sources, used for their radioactive properties. For this type of situation, the  
425 use of the source is understood, and as such the exposures can be anticipated and  
426 controlled from the beginning; and
- 427 • *Emergency exposure situations* are situations resulting from a loss of control of a source,  
428 or from intentional misuse of a source, which requires urgent and timely actions in order  
429 to avoid or mitigate exposure.

430 (33) The Commission considers human and environmental exposures resulting from  
431 industries involving NORM as existing exposure situations. The source is not deliberately  
432 introduced in the industrial process for its radioactive properties; it already exists in material  
433 used in the process or industry, and any protection decisions to control the exposure are made  
434 in that context. The process in which NORM in raw materials is concentrated, with changes  
435 of chemical-physical form resulting in production of radioactive release, residue and waste, is  
436 not for the purpose of introducing a new radioactive source; it is incidental even though it has  
437 to be managed. However, the Commission considers that when NORM is processed for its  
438 radioactive, fissile or fertile properties, it is a planned exposure situation.

439 (34) The philosophy of *Publication 103* (ICRP, 2007a) compared to *Publication 60*  
440 (ICRP, 1991) is to recommend a consistent approach for the management of all types of  
441 exposure situations. This approach is based on the application of the principle of optimisation  
442 using appropriate dose criteria. In existing exposure situations, because the source already  
443 exists when decisions on control are taken, the principle of application of dose limits is, a  
444 priori, not relevant. The relevant dose criteria is the reference level, and time may be needed  
445 to fully implement the protection strategy. For the protection of non-human species, the use  
446 of environmental reference levels based on Derived Consideration Reference Levels is also  
447 recommended (ICRP, 20014a). Whatever the type of exposure situation, however, the aim is  
448 to achieve a standard of protection that is proportionate to the level of risk.

449 (35) A graded approach, commensurate to the level of the risk as well as other  
450 considerations such as economic and societal, is appropriate and particularly relevant for  
451 industries involving NORM due to economic importance of industries, large volumes of  
452 residues and wastes and limited options for management, moderate level of doses, and  
453 potentially high cost of regulation in relation to reduction in exposure. Industries involving  
454 NORM are generally situations where multiple hazards and pollutants are present. The  
455 radiological risk may not be the dominant hazard, and consequently, there has often been no  
456 or only a limited radiological protection awareness. In such a context, the radiological  
457 protection system is not necessarily the driving force in safety. The graded approach should  
458 first take account of the existing knowledge and experience of these industries in the  
459 management of industrial hazards and then pragmatically integrate any additional measures  
460 necessary for the purposes of radiological protection.

461 (36) The doses resulting from the process in which NORM is concentrated are expected  
462 to remain relatively low whatever the circumstances. In the same way, the imaginable  
463 scenarios of loss of control of the radioactive material in industries involving NORM result in  
464 a limited impact in terms of doses and subsequent sanitary effects such as tissue reaction or  
465 immediate danger for life. Consequently, industries involving NORM present no real  
466 prospect of a radiological emergency, and thus are not likely to give rise to an emergency  
467 exposure situation, but releases and discharges may result in environmental damage.

### 468 3.1.2. Categories of exposure

469 (37) The Commission distinguishes between three categories of exposure: occupational,  
470 public and medical exposures. Occupational exposure is radiation exposure of workers  
471 incurred as a result of their work. However, because of the ubiquity of radiation, the  
472 Commission traditionally limits the definition of 'occupational exposures' to radiation  
473 exposures incurred at work as a result of situations that can reasonably be regarded as being  
474 the responsibility of the operating management. Medical exposure is the exposure of patients  
475 in the course of medical diagnosis and treatment. Public exposure encompasses all exposures  
476 other than occupational exposures and medical exposures of patients.

477 (38) Industries involving NORM can give rise to both occupational and public exposure,  
478 but not to medical exposure.

479 (39) In most cases the exposure of workers in industries involving NORM is adventitious  
480 because the presence of NORM in the material processed is a natural fact, without intentional  
481 addition for its radioactive purpose, and the workers are often not considered occupationally  
482 exposed. As indicated in *Publication 126*, referring to *Publication 65* (ICRP, 2014b, Para.  
483 59), workers who are not regarded as being occupationally exposed to radiation are usually  
484 treated in the same way as members of the public. Exposure of workers who are not  
485 considered occupationally exposed should anyhow be considered. In such case, it is the  
486 responsibility of the operating management to integrate the radiation risk among the others  
487 hazards and to address all the hazards in accordance with the agreed standards on health and  
488 safety at work.

489 (40) As described in section 4.1, a graded approach is recommended for the protection of  
490 workers in industries involving NORM, based on the selection of the reference level as well  
491 as the selection and the implementation of reasonable protective actions. This approach  
492 should also take into account, as explained in the previous sub-section, the integration of  
493 radiological protection in the procedures for the control of other hazards in a more global and  
494 synergistic way of hazard management.

495 (41) In rare cases, the level of dose or the application of special working procedures is  
496 needed for radiological protection purposes., In these cases, the measures recommended for  
497 occupationally exposed workers would apply (ICRP, 1997). The Commission's  
498 recommendations should not be interpreted as requiring all elements of a protection program  
499 irrespective of the circumstances. The approach should be graded, based on the hazards  
500 present.

501 (42) Public exposure is addressed through the control of NORM discharge, waste, residue  
502 (including their recycling and reuse) and possible legacy sites, as explained in section 4.2.

503 (43) Industries involving NORM generate environmental exposure through extraction,  
504 transportation, storage, processing, effluents, discharges and also from accidental releases. As  
505 indicated in section 4.3, environmental exposure is dealt with on the basis of common  
506 environmental standards, starting with an environmental impact assessment (EIA)  
507 considering the presence of NORM.

### 508 **3.2. Justification of protection strategies**

509 (44) The principle of justification is one of the two fundamental source-related principles  
510 that apply to all exposure situations. The recommendation in Para. 203 of *Publication 103*  
511 (ICRP, 2007a) requires, through the principle of justification, that any decision that alters the  
512 radiation exposure situation should do more good than harm. The Commission goes on to  
513 emphasise that for existing exposure situations, the justification principle is applied in  
514 making the decision as to whether to take action to reduce exposure and avert further  
515 additional exposures. Any decision will always have some disadvantages and should be  
516 justified in the sense that it should do more good than harm. In these circumstances, as stated  
517 in Para. 207 of *Publication 103* (ICRP, 2007a), the principle of justification is primarily  
518 applied in industries involving NORM when making the decision as to whether or not to  
519 implement a protection strategy for radiation exposures.

520 (45) As such, the justification falls under the ethical values of beneficence, which means  
521 promoting or doing good, and non-maleficence, which means avoiding causation of harm  
522 (ICRP, 2018), in order to reach the overall goal of societies to contribute to the well-being of  
523 individuals and the quality of the living together with the preservation of biodiversity and  
524 sustainable development.

525 (46) As explained in Para. 208 of *Publication 103* (ICRP, 2007a), the responsibility for  
526 judging the justification usually falls on governments or other national authorities to ensure  
527 that an overall benefit results, in the broadest sense, to society and thus not necessarily to  
528 each individual. However, input to the justification decision may include many aspects that  
529 could be informed by the industry involving NORM, workers, the public and organisations  
530 other than the government or national authority. As such, justification decisions could benefit  
531 from a stakeholder involvement process. In this context, radiological protection  
532 considerations will serve as one input to the broader decision-making process.

533 (47) The need for a protection strategy controlling exposure from NORM is better  
534 understood after radiological characterisation and taking into account health, economic,  
535 societal and ethical considerations. Since many industries involving NORM already exist, the  
536 Commission recommends the establishment at national level of a list of industries involving  
537 NORM for which a radiological risk assessment should be undertaken in order to determine  
538 if a protection strategy is justified. The level of control may then be determined through the  
539 implementation of the optimisation principle. If an ongoing industrial process involving  
540 NORM, not previously identified on a national list, appears to be of concern, the justification

541 of a protection strategy may be addressed on a case by case basis with the involvement of the  
542 relevant stakeholders.

543 (48) For industries involving NORM in the national list, when a new process using  
544 NORM is to be implemented, the principle of justification should be applied in the same way  
545 as for on-going processes, i.e. primarily when making the decision as to whether or not to  
546 implement a protection strategy for radiation exposures. Industrial processes will usually  
547 produce such significant economic and social benefits, and the radiological risks involved is  
548 unlikely to result in a decision that the NORM process, as a whole, would need to be  
549 considered unjustified. Exceptions can be dealt with on a case by case basis.

### 550 **3.3. Optimisation of protection**

551 (49) When a decision has been taken to implement a protection strategy, then the  
552 principle of optimisation of protection becomes the driving principle to select the most  
553 effective actions for protecting the exposed public, workers and the environment. It is defined  
554 by the Commission as the process to keep the magnitude of individual doses, the number of  
555 people exposed, and the likelihood of incurring exposures, as low as reasonably achievable  
556 (ALARA), guided by appropriate individual dose criteria, taking into account economic and  
557 societal factors. This means that the level of protection should be the best under the  
558 prevailing circumstances, adopting a prudent and reasonable attitude (see ICRP (2018)).

559 (50) To avoid serious inequity in the individual dose distribution, in line with the ethical  
560 value of justice (ICRP, 2018), the Commission recommends using individual dose criteria in  
561 the optimisation process (ICRP, 2007a, Para. 232). In addition to the reduction of the  
562 magnitude of individual exposures, a reduction of the number of exposed individuals should  
563 also be considered. The collective effective dose has been and remains a key parameter for  
564 optimisation of protection for workers, in comparing the exposures predicted from different  
565 options for protection strategies.

566 (51) The optimisation process should consider protection of the environment. The aim is  
567 to avoid deleterious effects on non-human species. Such approach should be commensurate  
568 with the overall level of risk and compatible with common standards of environmental  
569 protection, notably the optimisation of discharges in the environment. As is the case for  
570 human exposure, NORM processes may pose environmental risks from other constituents,  
571 and the radiological aspects have to be taken in an all hazard approach. In practice, the  
572 radiological impact should be included in the environmental impact assessment and  
573 monitored as necessary. The approach already developed by the Commission (ICRP, 2008,  
574 2014a), through a set of Reference Animals and Plants and numerical values for DCRLs is  
575 useful guidance when assessing possible deleterious radiation effects on non-human species,  
576 their diversity, communities and ecosystems in general. The results contribute to decisions on  
577 the most appropriate option for controlling the source.

578 (52) In case of industries involving NORM, the optimisation process is implemented in  
579 generally the same way as for other industries. However, because of the prevailing  
580 circumstances and notably since in some circumstances the source cannot be controlled in the  
581 way it is for other sources, the options to reduce doses may be more limited and/or may  
582 require different resources. Such challenges suggest the need for flexibility and  
583 reasonableness in the implementation of the optimisation process.

584 (53) The involvement of relevant stakeholders early in the optimisation process will  
585 contribute to the transparency of the process and increase confidence in the outcome.

586 **3.3.1. Dose criteria**

587 (54) The Commission recommends the use of reference levels as dose criteria in existing  
588 exposure situations. The reference level represents the value of dose used to guide and drive  
589 the optimisation process. The selection of the reference level should consider the actual  
590 individual dose distribution, with the objective of identifying those exposures that warrant  
591 specific attention. Reference levels are guides for selecting amongst protective options in the  
592 optimisation process in order to maintain individual doses as low as reasonably achievable  
593 taking into account economic and societal factors, and thus prevent and reduce inequities in  
594 the dose distribution. Reference levels are also benchmarks against which the results of  
595 protective actions can be judged to determine if protection is reasonably optimised and  
596 effective.

597 (55) For existing exposure situations, the Commission recommends setting reference  
598 levels typically within the 1 to 20 mSv per year band as presented in Table 5 of *Publication*  
599 *103* (ICRP, 2007a), and with the possibility that the reference level could be lower than 1  
600 mSv per year. The 1 to 20 mSv per year band presupposes that the sources or the pathways  
601 can generally be controlled, and individuals receive direct benefits from the activities  
602 associated with the exposure situation, but not necessarily from the exposure itself. However,  
603 the selection of the reference level for a particular exposure situation should be made based  
604 upon the characteristics of the circumstances (ICRP, 2007a, Para. 234), considering the  
605 individual dose distribution, with the objective to identify those exposures that warrant  
606 specific attention and meaningfully contribute to the optimisation process. Industries  
607 involving NORM generally give rise to low levels of individual exposure and the appropriate  
608 reference level would in most cases be less than a few mSv per year. The selected reference  
609 level should be meaningful for protection purposes, not a generic value which would not help  
610 to identify individuals for whom some further consideration might be needed. Thus,  
611 according to the characteristics of the exposure situation, notably the actual and potential  
612 exposure pathways, the individual dose distribution and the prospect for optimisation, an  
613 appropriate reference level can be selected, either below a few mSv per year or above a few  
614 mSv if necessary, but very rarely will a reference level exceeding 10 mSv per year be  
615 necessary.

616 (56) Chapter 4 contains specific bands of reference levels that are recommended for the  
617 protection of NORM workers and the public, respectively. They are consistent with the  
618 approach recommended in *Publication 103*.

619 (57) The principle of individual dose limits applies only in planned exposure situations  
620 (ICRP, 2007a, Para. 203). In the case for NORM exposure, following characterisation of the  
621 situation, and optimisation of protection with reference levels, the protection program becomes  
622 established, with controls that are effective. The magnitude of exposures will often be  
623 relatively low, reflecting the optimisation of protection with reference levels.

624 (58) The Commission recognises that some authorities have already specified the  
625 application of dose limits for some industries involving NORM. Such use is understandable,  
626 as a limit is frequently used as one regulatory mechanism to judge the acceptability of a  
627 radiation control program. Such a use is not unacceptable in circumstances when the source is  
628 well characterised, and the control programs have been established. However, specifying a  
629 limit for regulatory purposes is not meant to imply that the situation has been, or needs to be,  
630 transformed into a planned exposure situation. In the vast majority of industries involving  
631 NORM, the application of 'limits' expressed in terms of dose provides no real additional  
632 protection for workers, and may entail administrative burdens that are not in keeping with  
633 efficient and effective use of resources.

634 **3.3.2. The optimisation process**

635 (59) Optimisation of protection of the human health and the environment in existing  
636 exposure situations is implemented through a process that involves (a) the assessment of the  
637 exposure situation; (b) identification of the possible protective options to maintain or reduce  
638 the exposure to as low as reasonably achievable taking into account economic and societal  
639 factors; (c) the selection and implementation of the most appropriate protective options under  
640 the prevailing circumstances; and (d) the regular review of the exposure situation to evaluate  
641 if there is a need for corrective actions, or if new opportunities for improving protection have  
642 emerged.

643 (60) In this iterative process, the Commission considers that the search for equity in the  
644 distribution of individual exposures is an important aspect (ICRP, 2006). It should be noted  
645 that, in industries involving NORM, the distribution of individual doses for both workers and  
646 members of the public may be very large. The protection efforts should focus individuals on  
647 the higher dose tail of the distributions, i.e. on the most exposed individuals, so as to  
648 determine if efforts are reasonable to reduce their exposures, while simultaneously trying to  
649 reasonably reduce the exposure of the whole exposed population.

650 (61) The decision making for control of industries involving NORM should be open and  
651 transparent. Stakeholders should be involved as necessary, including the workers, community  
652 and others as appropriate. Their concerns and ideas should be listened to and taken into  
653 account. A transparent system for decision making will allow for controversial issues to be  
654 properly addressed and resolved, although not necessarily with full agreement from all  
655 parties.

656 (62) The inclusion of natural or man-made radiation highlights the need to foster the  
657 development of an appropriate radiological protection culture within the organisation and  
658 community, enabling each individual to make well-informed choices and behave wisely in  
659 situations involving potential or actual exposure to ionising radiation (ICRP, 2006). It is a  
660 matter closely tied to the ethical concept of dignity (ICRP, 2018).

661 (63) Detailed advice of the Commission on how to apply the optimisation principle in  
662 practice has been provided earlier (ICRP, 1983, 1990, 1991, 2006), and remains valid.  
663

664 **4. IMPLEMENTATION OF THE SYSTEM OF RADIOLOGICAL**  
665 **PROTECTION TO INDUSTRIAL PROCESSES INVOLVING NORM**

666 **4.1. Protection of workers**

667 **4.1.1. General considerations**

668 (65) Typical industries involving NORM process a wide range of raw materials and  
669 activity concentrations, and radiation exposure is adventitious, as the processes are not in any  
670 way intended to take advantage of the radioactive materials. Depending upon the  
671 circumstances, it may not be necessary to consider controls directly applicable to a particular  
672 individual in order to properly control exposures. This does not mean that protection is not  
673 warranted, but that the control is exercised on the workplace and the conditions of work  
674 rather than on the worker her/himself. It is not easy to define criteria applicable in all  
675 situations. Thus, a graded approach for the protection of workers is recommended.

676 (66) Three main exposure scenarios have been identified:

- 677 • Exposure to large quantities of material as an ore or a stockpile of raw material;
- 678 • Exposure to small quantities of material with concentrated radionuclides such as mineral  
679 concentrates, scales and sludge;
- 680 • Exposure to material that has been volatilised in high temperature processes, like slag,  
681 precipitator dust and furnace fume.

682 (67) The main exposure pathways for work with NORM are:

- 683 • External exposure (mostly due to gamma radiation, but occasionally beta radiation  
684 exposure to the lens of the eye and to the skin may also need to be considered);
- 685 • Internal exposure from inhalation dust and to a much lesser extent ingestion of  
686 radioactive dust, as well as exposures due to radon gas and its progeny, which can occur  
687 above ground or underground (e.g., the build-up of radon gas in underground  
688 workplaces) and sometimes thoron emanating from NORM. In practice, radon emanating  
689 from such materials is often indistinguishable from that already present (e.g. from the  
690 ground).

691 (68) The Commission considers that radon and thoron in the workplace, irrespective of  
692 the source, should be managed as a single source, i.e. as described in *Publication 126*. That  
693 Publication recommends an integrated approach for protection against radon exposure in all  
694 buildings, whatever their purpose and the status of their occupants. The strategy of protection  
695 in buildings, implemented through a national action plan, should be based on application of  
696 the optimisation principle using a reference level translated for practical reasons to  
697 concentrations in air, to facilitate implementation. The Commission recommends that  
698 national authorities to set a derived reference level that is as low as reasonably achievable in  
699 the range of 100 to 300 Bq m<sup>-3</sup> taking the prevailing economic and societal circumstances  
700 into account. The corresponding effective dose depends on a number of factors such as  
701 breathing rates (see ICRP (2017)). As described in *Publication 126*, if Radon mitigation  
702 actions cannot reduce levels to less than the reference level, the exposure will need to be  
703 considered as part of the occupational exposure.

704 (69) It is important to note that workers in industries involving NORM are exposed to  
705 radiation and also to other hazards. The radiological risk is often not the dominant hazard,  
706 and may historically not even have been a consideration. In such a context, there is often a  
707 lack of radiological protection awareness or a culture supporting such protection. However,  
708 such industries do have experience and expertise in the management of occupational health

709 and safety, and there is an opportunity to build a radiological protection culture in an  
710 integrated fashion. In many cases, actions to reduce workplaces hazards such as airborne  
711 dust, will also restrict radiation exposures, and an integrated approach to worker protection is  
712 recommended.

713 (70) Protection of workers in industries involving NORM should be based on a graded  
714 approach to control radiation exposures, according to the annual effective dose (due to the  
715 activities involving NORM) that is likely to be received and the scope for dose reduction that  
716 may be necessary.

717 (71) In practice, a graded approach can be realised through the selection of suitable dose  
718 reference levels, the selection of the requisites, i.e. appropriate protective actions, and the  
719 integrated implementation of these requisites. The practical implementation of this approach  
720 will also help to determine whether or not the workers should be considered as  
721 occupationally exposed to radiation.

722 (72) This approach can also serve as the basis for creating a common understanding  
723 between regulatory authorities and other stakeholders such as operators, workers and their  
724 representatives, as well as health, safety and environmental professionals, of the radiological  
725 aspects of the various processes involved and the ways in which these aspects can be  
726 addressed reasonably and effectively.

#### 727 **4.1.2. Selection of the dose reference level for workers**

728 (73) Since the industries involving NORM are so diverse, there is no unique numerical  
729 value which is appropriate as a reference level for all of them. According to the  
730 characteristics of the exposure situation, notably the actual and potential exposure pathways,  
731 the individual doses distribution and the prospect for optimisation, the appropriate reference  
732 level can be selected:

- 733 • Below a few mSv per year for most cases, and
- 734 • Above a few mSv, but very rarely exceeding 10 mSv per year, when necessary because  
735 of the circumstances involved.

736 (74) Considering the current information about the dose distribution of doses of workers  
737 in many industries involving NORM, the selection of a reference level above 10 mSv per  
738 year would not be necessary in terms of radiological protection.

739 (75) As indicated above, these doses exclude exposures from radon or thoron.

740 (76) In most situations, the residual doses are not expected to exceed the reference level,  
741 particularly after the effective implementation of protective measures. The reference level  
742 remains useful to allow judgement on the appropriate functioning of the program, and to  
743 indicate if modifications of the program are needed.

#### 744 **4.1.3. Selection and implementation of requisites**

745 (77) When considering measures to optimise exposures to NORM workers, the starting  
746 point should always be the existing industrial safety and hygiene controls, i.e. for non-  
747 radiological hazards in the workplace. Experience shows that a well-managed, safety-focused  
748 workplace will already have done much to reduce radiation exposures from NORM. Where  
749 additional radiological protection controls are considered necessary, as far as practicable  
750 these should be integrated into the wider safety strategy.

751 (78) The strategy for protection of workers as defined in Conventions from the  
752 International Labour Organisation (Convention 167, Convention 176), comprises three main  
753 steps:

- 754 (a) Eliminate the risk, for example by replacing hazardous substances by harmless or less  
755 hazardous substances wherever possible;
- 756 (b) Minimise the risk, for example by technical measures applied to the plant, machinery,  
757 equipment or process;
- 758 (c) In so far as the risk remains, undertake other effective measures related to the workers  
759 themselves, such as the use of personal protective equipment.

760 (79) The same scheme is relevant for the protection of workers in industries involving  
761 NORM. Control of the workplace and the conditions of work are to eliminate or minimise the  
762 risk, while the control of individuals enters when adequate protection has not already been  
763 achieved. Moving from controls of the workplace to individual controls needs to be carefully  
764 considered as these controls are costly, and the preference would be to have sufficient controls  
765 for the workplace so that individual controls are not needed. The requisites related to the  
766 workplace and the conditions of work, are described below.

767 (80) *Characterisation of the situation*: this characterisation – determining who is  
768 exposed, when, where and how – is an important starting point for the protection of workers.  
769 It includes the characterisation of the source, with the aim of identifying the distribution of  
770 NORM radionuclides and their activity concentrations throughout the industrial process,  
771 including mode of exposure, chemical and physical characteristics of particulates, NORM  
772 distribution and activity concentrations at all stages of the industrial process. Feed materials,  
773 intermediates, residues and wastes (including contamination of the plant), and discharges to  
774 the environment should be considered.

775 (81) Characterising the source will help identify the main exposure pathways to workers,  
776 the public and the environment. In terms of exposure to workers, the next step is to  
777 characterise exposed groups or individuals and make an initial assessment of the annual doses  
778 (effective doses arising from external exposure and internal exposure through inhalation)  
779 received from the work with NORM.

780 (82) The characterisation of the exposure situation may, of course, vary in detail  
781 according to the prevailing circumstances involved. In practice, external gamma radiation and  
782 internal exposures from radioactive dust inhalation are the two exposure pathways of interest  
783 (plus radon which is addressed separately). When considering the likely annual radiation  
784 exposure of workers in different industries involving NORM, it is important that these are  
785 based on realistic estimates, i.e. taking into account actual external radiation and airborne  
786 contamination levels in the workplace and actual working patterns and procedures. When  
787 estimating radiation exposures, the effect of existing occupational health and safety (OHS)  
788 provisions should be taken into account (e.g. industrial hygiene, industrial safety, workplace  
789 controls on airborne dust).

790 (83) It is important that this characterisation stage is fully documented, so as to provide a  
791 sound basis for any future decision-making.

792 (84) The characterisation will form the basis for the justification of the protection  
793 strategy, notably the need for specific requisites for radiological protection purposes, as well  
794 as for the scaling of the optimisation process.

795 (85) The initial characterisation should be subject to periodic review. The detail and  
796 frequency of this periodic review should be commensurate with the level of risk. When  
797 feedstocks, ores, production practices or other factors that can affect dose are expected to  
798 change significantly, a new characterisation should be undertaken.

799 (86) *Obtaining expert radiological protection advice*: such expertise is normally required  
800 from the beginning, i.e. to assist with the characterisation of the exposure situation.  
801 Typically, industries involving NORM have operated for many years before the issue of  
802 natural radioactivity has been addressed. As a result, there is often a complete lack of

803 knowledge about radioactivity and radiological protection. Consequently, the first step should  
804 be to seek expert advice on this issue, even where industries involving NORM already have  
805 their own technical support in a wide range of other areas. Such specific expertise can be  
806 provided either internally or by external consultants. Such radiological protection expertise  
807 should be sought by both operating management, and also by the national authorities where  
808 no specialised expertise exists. The need for advice from a radiological protection expert may  
809 be temporary (e.g. where it can be shown from an initial review and assessment that  
810 exposures are very low), or may be required on an on-going basis.

811 (87) *Initial actions to prevent or reduce the hazard*: this corresponds to the first step of  
812 the ILO approach. At the initial stage, it is useful to consider if there are any ways in which  
813 the hazards from NORM can be either eliminated from the process, or else substantially  
814 reduced. Examples include the selection of alternative feed materials (i.e. with much lower  
815 concentrations of NORM), or changes to the process designed to prevent the accidental  
816 accumulation or concentration of radionuclides. Whilst recognising that this is likely not  
817 practical or possible, particularly in long standing industries involving NORM, it nevertheless  
818 should be given some consideration.

819 (88) *Delineation of areas*: the delineation of areas is a well-established element of the  
820 control strategy in planned exposure situations. However, it is also part of a wider industrial  
821 health and safety strategy, i.e. to identify areas where additional safety measures (e.g.  
822 working procedures, ventilation requirements, use of personal protective equipment,  
823 limitation of access) are required. To be effective, area delineation requires warning signs  
824 and, in some cases, formal restrictions on access. The same approach is appropriate for  
825 industries involving NORM. Worker right-to-know protocols may determine the type of  
826 signage needed. The concept may already be in place in some industries, as there would, for  
827 example, be warnings and controls for dust.

828 (89) *Engineered controls*: as previously said, the characteristics of NORM are such that  
829 scenarios involving high doses from accidental exposures do not generally exist. Thus, the  
830 traditional engineering controls to prevent such exposures are not required. Instead, measures  
831 to restrict chronic exposures from NORM should be considered. These start with the design  
832 and layout of the facility, and then specific measures to control dust, such as containment and  
833 ventilation. Industries involving NORM such as mineral processing plants can be very dusty,  
834 and a dust control strategy and programme should already be in place in such facilities.  
835 Improvements to containment and ventilation systems should be considered holistically, i.e.  
836 in terms of their overall effect on radioactive and other materials.

837 (90) Specific engineering measures to restrict external radiation exposures (i.e. shielding)  
838 may be required: for example, local shielding around pipes and vessels containing NORM at  
839 very high activity concentrations may be considered. More commonly, however, protection is  
840 provided through adjustments to working patterns and, in some cases, relocation of materials,  
841 plant or persons (distance).

842 (91) *Working procedures*: these procedures, such as limiting time of exposure, can be  
843 very effective in restricting both internal and external doses, even where exposures are  
844 already low. Often, all that is required is observance of good industrial hygiene and simple  
845 safe working procedures, supported by an appropriate amount of training (see below) and  
846 supervision.

847 (92) The requisites listed above, complemented by at least a general information  
848 programme for workers (see below), may be sufficient for the protection of workers in most  
849 industries involving NORM. However, they can be complemented, as necessary, by  
850 requisites related to the individuals.

851 (93) *Information, instruction and training*: the information and training provided to  
852 workers should be proportionate to the radiation risk and the precautions that need to be  
853 taken. There is a basic need to share information and generally raise awareness about NORM  
854 within the workplace. Information should in particular be provided to pregnant and  
855 breastfeeding workers. NORM workers are key stakeholders in this process, and the  
856 principles of open communication and engagement should be applied at an early stage.  
857 Where special precautions to restrict exposures to radiation are required, the relevant workers  
858 should receive specific training to understand the nature of the radiological risks and the  
859 importance of protective actions, and practical instructions on how to implement these  
860 actions.

861 (94) *Personal protective equipment (PPE)*: this includes protective clothing and  
862 respiratory protective equipment (e.g. dust masks), and these are already widely used in  
863 NORM workplaces to protect against other hazards. PPE should be selected with due  
864 consideration of the hazards involved. The equipment should not only provide adequate  
865 protection but also be convenient and comfortable to use. The effectiveness of any existing  
866 PPE should be assessed before determining whether improved or additional PPE for  
867 radiological protection purposes is required. Engineered controls are the favoured option,  
868 with working procedures and, finally, protective respiratory equipment being considered only  
869 where further engineering controls are not effective or practicable. Consideration should also  
870 be given to the possibility of an increase in exposure caused by the additional constraints of  
871 the personal protective equipment.

872 (95) *Dose assessment*: an assessment of the exposure of workers is required as part of the  
873 initial characterisation described above. It is envisaged that this will be based on workplace  
874 measurements and other information (e.g. about the process and working practices), rather  
875 than individual dosimetry. In practice, although the level of dose may not be the only  
876 criterion, where worker doses are estimated to be higher than a few mSv per year, an ongoing  
877 programme of dose assessment should be implemented, according to a graded approach.  
878 Where doses are above a few mSv per year, it is expected that they will be estimated on the  
879 basis of workplace measurements. Individual dose assessment, for example through the use  
880 of personal dosimeters, may be useful as a means of providing information to help optimise  
881 exposures, but is not expected to be undertaken on a routine basis.

882 (96) Where doses are well above a few mSv per year, there is a need to undertake  
883 individual dose assessments. For external radiation, this should be done with personal  
884 dosimeters (passive or electronic). Assessment of internal exposures from dust inhalation is  
885 much more challenging; however, in very dusty NORM workplaces there may already be a  
886 dust monitoring programme which can be adapted to also provide estimates of radiation dose.  
887 If not, and if internal doses are high, arrangements with a suitable internal dosimetry service  
888 will need to be considered. It should be noted, however, that such exposures are unlikely to  
889 be considered optimised, and that suitable protective actions should be more than capable of  
890 reducing internal exposures.

891 (97) *Dose recording*: both workplace and individual data related to the estimation and  
892 assessment of worker doses should be recorded and kept for sufficient time. The recording  
893 may be carried out in different ways according to the situation. For instance, it could be by  
894 keeping track of ambient exposure in a given place of work and of people who frequented it,  
895 so as to be able to assess the doses of a given worker retrospectively if necessary. It could  
896 also be carried out by registering individual doses in the dedicated sheet in the medical record  
897 of each concerned worker.

898 (98) *Health surveillance*: in some industries involving NORM there is already a health  
899 surveillance programme for non-radiological reasons. It is considered unlikely that health

900 surveillance specifically for radiological protection purposes will be required, except in a  
901 very few cases where annual doses well above a few mSv per year are repeatedly received. If  
902 this is the case, then it is expected that existing provisions for the health surveillance of  
903 workers occupationally exposed to radiation will be used, and will be sufficient.

904 (99) Most of these requisites can be implemented more or less thoroughly. Workers are  
905 likely to be considered as occupationally exposed when, despite all reasonable efforts to  
906 reduce exposure, elevated individual doses persist and when the application of special  
907 working procedures are needed to perform the job. In such cases, education and training,  
908 individual radiation dose monitoring and recording, or health surveillance for radiological  
909 protection purposes may all need to be implemented as described in ICRP *Publication 75*  
910 (ICRP, 1997).

## 911 **4.2. Protection of the public**

912 (100) The general approach to the protection of the public should also start with a  
913 characterisation of the exposure situation (who is exposed, when, where and how), including  
914 analysis of exposure pathways and dose assessments. This characterisation forms the basis  
915 for the justification of a protection strategy. Then the optimisation process should be  
916 implemented, including the selection of a reference level, the selection and the  
917 implementation of the protective actions, the involvement of stakeholders in the decision-  
918 making process and the provision of a long-term monitoring of the situation if necessary.

919 (101) This process should be implemented in a reasonable way, keeping in mind the  
920 ethical values of beneficence/non-maleficence, prudence, justice and dignity. In more  
921 complex situations, working with stakeholders to identify their underlying interests for each  
922 ethical value can be very useful in working towards an acceptable and sustainable solution.

923 (102) The reference level for the protection of the public should be selected below a few  
924 mSv per year. In some cases of public exposure for industries involving NORM, a reference  
925 level less than 1 mSv per year may, in fact, be the most appropriate taking into account the  
926 distribution of doses that exists. The protection of the public should be addressed as a whole,  
927 i.e. taking into account the different pathways. In a given situation, the pathways need to be  
928 considered with respect to NORM discharge, waste, residue and possible legacy sites. In  
929 practice, the most exposed individuals to each pathway belong to different groups so that the  
930 reference level can generally be applied to any given pathway. The reuse and recycling of  
931 NORM residues may be starting point of a new NORM process.

### 932 **4.2.1. Discharges from industries involving NORM**

933 (103) Liquid and gaseous radioactive and/or non-radioactive effluents may be deliberately  
934 discharged from the normal operation of industries involving NORM. Radionuclides may  
935 also be precipitated onto particles in the stream of liquid or gaseous effluents (aerosols). In  
936 certain cases, such as oil and gas extraction, the phosphate processing industry and the  
937 combustion of coal, NORM discharges have been an issue for the protection of both people  
938 and the environment. Therefore, effluents should be properly controlled taking into account  
939 the radiological and non-radiological impacts and, if necessary, restricted in order to protect  
940 the public and the environment.

941 (104) A comprehensive site-specific control of the discharge should, from a radiological  
942 protection point of view, include the following steps:

943 (a) Radiological characterisation of discharge;

- 944 (b) Identification of potential exposure pathways and radionuclides mobility;  
945 (c) Dose assessments and risk estimation;  
946 (d) Justification of measures to control discharge;  
947 (e) If so, selection of a reference level, and;  
948 (f) Selection and implementation of measures within a protection strategy through an  
949 optimisation process (ALARA).

950 (105) The protection strategy should include preventive actions aimed at eliminating or  
951 reducing the quantity and the concentration of discharges, as well as mitigation actions  
952 aiming at reducing the impact of the discharge in term of public and environmental  
953 exposures. The optimisation process and the involvement of stakeholders are case specific  
954 and depend, in practice, on the operational characteristics of the NORM facility, discharge  
955 processes, radioactivity levels and estimated risk, the public groups involved, as well as  
956 societal and political aspects and public awareness. Optimisation in practice can be complex  
957 due to the fact that some processes such as effluent treatment may lead to the production of  
958 further waste in which there are increased concentrations of radionuclides, or else produce an  
959 increase in the overall volume of waste produced.

960 (106) Attention should also be paid to the issue of drinking water, to the environmental  
961 impact (see below), current and future land use in the area and to the possible presence of  
962 several facilities in the same area.

963 (107) The use of reference levels translated into a measurable quantity (for example, in  
964 terms of total activity and/or activity concentration) may be appropriate for industries  
965 involving NORM.

#### 966 **4.2.2. Waste**

967 (108) Waste, both liquid and solid, is material with no further planned use. Industries  
968 involving NORM can produce wastes containing both radioactive and non-radioactive  
969 pollutants: both should be managed consistently. Globally, industries involving NORM  
970 produce waste ranging from small volumes of waste with high concentrations of  
971 radionuclides to large volumes of waste with low concentrations of radionuclides.

972 (109) All waste should be characterised in order to determine the proper methods for  
973 disposal. Waste treatment should be considered and performed as relevant in the optimisation  
974 process, although concentration of wastes to high levels can pose challenges. The issue of  
975 waste should be considered from its generation to final disposal when starting or designing a  
976 new project ('from cradle to grave').

977 (110) The method of disposal of NORM waste should be proportionate to the type and the  
978 level of hazard taking into account all types of pollutants in the presence (radioactive and  
979 non-radioactive). Depending on level of radioactivity and volume of waste, a graded  
980 approach should apply. Some waste could be treated as industrial or hazardous waste and  
981 disposed of accordingly in near surface landfills. The disposal of waste with higher  
982 concentrations of radionuclides should be consistent with the management of radioactive  
983 waste.

#### 984 **4.2.3. Residues**

985 (111) Residues are materials which can be recycled and re-used. They are mainly coming  
986 from upstream of the NORM cycle (exploration, extraction of material) and the activity  
987 concentration in the residues may be significantly enhanced compared to the raw material.  
988 Like waste, they should be characterised and properly stored before potential reuse. There are

989 economic and ecological arguments for finding a use for NORM residues. By-products and  
990 residues of a given industry involving NORM can be used as feedstock by other industries  
991 involving NORM, as land-fill (if there are no chemical hazards or pathways to groundwater),  
992 and/or in commodities (e.g. building materials). Using residues as feedstock may be the  
993 starting point of a new NORM process. Recycling or re-use helps to reduce waste volumes.  
994 However, in some cases, it could result in exposure of workers, the public and the  
995 environment. Residues that are stockpiled for any length of time should be properly managed  
996 to prevent environmental contamination.

997 (112) The implementation of a protection strategy should be considered for reuse or  
998 recycling of NORM residues. The assessment should take account of various elements such  
999 as the level of exposure, the pollution of the environment, the alternatives, the future of the  
1000 products or the societal acceptance. In rare cases, based on this assessment, the new process  
1001 may not be justified and the residues may need to be treated as waste.

1002 (113) When a protection strategy is justified, optimisation should be considered  
1003 recognising that the scope for dose reduction may be limited.

#### 1004 4.2.4. Building materials

1005 (114) Building materials may contain natural radionuclides originating from raw materials  
1006 (e.g. extracted from quarries) or residues from industries involving NORM or a mixture of  
1007 materials some of which are naturally radioactive (e.g. concrete). They can cause public  
1008 exposures by direct external gamma radiation and by releasing radon into indoor air.  
1009 Occupational exposures in the manufacture and handling of building materials are usually  
1010 low but they should be managed in a graded approach as in any other industry involving  
1011 NORM.

1012 (115) The reference level for building materials should be of the order of 1 mSv y<sup>-1</sup>, or  
1013 less, expressed as effective dose caused by external gamma radiation to members of the  
1014 public. A reference level of this order should also ensure that any radon exhalation from  
1015 <sup>226</sup>Ra in building materials is unlikely to be the cause for the reference levels set for indoor  
1016 radon concentration to be exceeded. The exhalation of thoron is not expected to be of  
1017 concern. Radon exposures should be managed in line with *Publication 126* (ICRP, 2014b)  
1018 irrespectively of its origin.

1019 (116) Lists of building materials, raw materials and residues of concern may be found in  
1020 various publications (EURATOM, 2013; IAEA, 2015). There are also different  
1021 methodologies for screening building materials of concern and for assessing the dose caused  
1022 by building materials (EC, 1999b; IAEA, 2005; EURATOM, 2013). A common screening  
1023 method is the use of an activity concentration index derived from the reference level, the  
1024 value of which is calculated on the basis of the concentrations of <sup>226</sup>Ra, <sup>232</sup>Th and <sup>40</sup>K. Where  
1025 the value of the index is less than 1, the dose level of 1 mSv per year cannot be exceeded  
1026 under any circumstances. Because of its very conservative assumptions, the index does not,  
1027 however, provide information on the actual exposure caused by a building material. For  
1028 assessing the dose, more elaborate methods need to be used in order to consider the actual  
1029 concentrations and locations of a certain building material in a building (EC, 1999b;  
1030 EURATOM, 2013; IAEA, 2015).

1031 (117) A protection strategy should be established with the aim to promote building  
1032 materials that do not exceed the reference level. The strategy may encompass measures such  
1033 as providing information on the levels of exposure caused by different building materials,  
1034 labelling of materials, suggesting the use of materials with low radioactive concentration or  
1035 limiting the use of certain materials causing significant exposures. In keeping with the ethical

1036 value of beneficence/non-maleficence, it is important to ensure that the measures envisaged  
1037 are actually reasonable and feasible before deciding on them.

1038 (118) Special attention should be paid to processes where residues with exceptionally high  
1039 activity concentrations are incorporated into building materials. They should not be  
1040 implemented for the purpose of intentional diluting or for bypassing more stringent  
1041 requirements on the appropriate management of such residues. This applies irrespective of  
1042 whether the reference level for building materials might be exceeded.

1043 (119) There may be a need to apply a similar approach for other construction materials  
1044 such as those used for foundations of houses, surfaces of yards, playgrounds, streets and  
1045 roads, as well as, bridges and other similar structures. Dose assessments and separate derived  
1046 activity concentration indexes may need to be considered.

#### 1047 **4.2.5. Legacy sites**

1048 (120) Industries involving NORM account for many current legacy sites with radioactive  
1049 contamination. NORM legacy sites have been identified more frequently with the rising  
1050 awareness of industries involving NORM and related radiological protection issues. This  
1051 situation shows that the dismantling of facilities when shutdown is sometime not sufficiently  
1052 considered from a radiological protection point of view. Technologies and methods already  
1053 exist and should be implemented in order to avoid legacy sites.

1054 (121) The issue of legacy sites is in the scope of a future ICRP Publication in preparation,  
1055 therefore the present publication provides only a few general considerations. The assignment  
1056 of responsibility or liability for maintenance and remediation of old legacy sites may be an  
1057 issue due to the elapsed time and often lost information. Sites with no known responsible  
1058 party are often called orphan sites. New legacy sites should be avoided through a proper  
1059 dismantling of the industries involving NORM and durable administrative control if  
1060 necessary.

1061 (122) The justification of the remediation of legacy sites is not only driven by radiological  
1062 protection considerations. As in active industries involving NORM, other hazards such as  
1063 heavy metals may also be present. The reference level should be in the lower range of the  
1064 band 1-20 mSv  $y^{-1}$ . The reference level is not the endpoint of the remediation. The endpoint  
1065 should be an optimised level of dose below the reference level, determined on a case by case  
1066 basis taking into account the prevailing circumstances (including the situation pre-  
1067 disturbance), the future use of the site (when it can be predicted) and the possible conditions  
1068 (or restrictions) of use.

1069 (123) The implementation of the optimisation principle is often a challenge, for example  
1070 because it is occasionally difficult to make a distinction between NORM contamination and  
1071 the natural background radioactivity. The challenge may also be due to a lack of societal  
1072 acceptance. The involvement of stakeholders in the decision process is of great importance  
1073 for the management of legacy sites.

1074 (124) The workers involved in the remediation process may need to be specifically trained  
1075 for working with radiation. As such, they should be considered as occupationally exposed.

1076 (125) If common workers or members of the public are participating in the remediation (in  
1077 their home or in places open to the public), relevant information and recommendations  
1078 should be communicated to them as well as some protective equipment, such as dust masks,  
1079 as relevant.

1080 **4.3. Protection of the environment**

1081 (126) Large quantities of NORM may be present in the environment in form of mixed  
 1082 material together with other contaminants. Through the time, different geochemical and  
 1083 physical processes in the environment disturb the NORM radionuclides equilibrium. It is well  
 1084 known that mechanisms such as selective dispersion, leaching and transfer, fractionation,  
 1085 bioaccumulation, and reaction with other contaminants can result in changes in  
 1086 environmental impact over time. In this kind of environmental exposure, it can be difficult to  
 1087 use a simple approach for risk assessments to evaluate the possible risk and effects for the  
 1088 non-human species.

1089 (127) The optimisation process should address the protection of the environment, i.e. the  
 1090 protection of non-human species and not only the prevention of exposure of humans through  
 1091 environmental pathways (ICRP, 2007a). Mechanisms to control releases of effluents, in  
 1092 particular, can be informed by the prediction of dose for non-human biota. The selected  
 1093 controls, may, or may not, be specifically driven by radiological protection for non-human  
 1094 species, but the relative contribution for different options is useful information. However, the  
 1095 information on elevated NORM activity concentration in the certain environmental  
 1096 compartment does not necessarily mean effects in non-human species, and the assessment of  
 1097 impact must consider a variety of factors beyond just the estimated dose.

1098 (128) Over last decades, considerable international and national efforts have been made to  
 1099 develop an approach for radiological protection of the environment. To raise awareness about  
 1100 radioactivity in industrial activities has become important at both national and international  
 1101 levels. Industries involving NORM have been generally following common standards to  
 1102 protect the environment from other pollutants than radioactivity. Depending on the legal  
 1103 requirements, an environmental impact assessments (EIA) may be performed to demonstrate  
 1104 compliance with environmental standards. Radiological impact from NORM should be  
 1105 included in an EIA. In situations where there is not the requirement to perform an EIA, a  
 1106 specific assessment for NORM should be considered including both radiological and non-  
 1107 radiological impact and provide an input to decisions on controls.

1108 (129) The EIA should consider the total impact of NORM activity, which for the specific  
 1109 purpose of protecting the environment from the harmful effects of radiation entails:

- 1110 (a) Radiological characterisation of NORM discharge, including the data on background  
 1111 NORM levels;
- 1112 (b) Identification of environmental pathways and mobility of radionuclides;
- 1113 (c) Analyses of key non-human species uptake;
- 1114 (d) Modelling and evaluation of potential radiation effects to doses by using the approach  
 1115 with Reference Animals and Plants (RAPs), Representative Organism and the  
 1116 corresponding bands of derived consideration reference levels (DCRL), or specific  
 1117 environmental reference levels derived for the purpose of the assessment (ICRP, 2008,  
 1118 2014b);
- 1119 (e) Risk estimation, taking into account the actual species present or likely to be present, and  
 1120 management using the appropriate reference levels to inform optimisation decisions.

1121 (130) For radiological characterisation of NORM released in the environment, it is  
 1122 necessary to perform the site-specific analysis of radionuclides with respect to their physical  
 1123 and chemical forms and activity concentrations in source, but also at environmental media of  
 1124 concern (air, water, sediment, soil). To be able to assess exposure of non-human species, it is  
 1125 further necessary to identify the mobility of radionuclides, their spatial and temporal  
 1126 variation, environmental pathways to plants and animals and their bioavailability. An  
 1127 approach with reference animals and plants (RAPs) and derived consideration reference

1128 levels has been developed (ICRP, 2008, 2014b). Dosimetry models to calculate specific  
1129 exposure doses from chosen radionuclides and for ecosystems and organisms of concerns  
1130 have been available for site-specific use. A degree of precaution may be considered necessary  
1131 because of the importance of the site or habitat, or the importance of the actual species  
1132 present or likely to be present. It is important to note that, in many cases, other constituents  
1133 are which present hazards to plants and animals will also be present. The Commission  
1134 emphasises its recommendations that an all hazard approach be undertaken.

1135 (131) Regarding dose criteria for protection of non-human species, risk characterisation  
1136 and proper optimisation, bands of environmental derived consideration reference levels can  
1137 be considered as reference dose rates intervals within which there is some chance of  
1138 deleterious effect from ionising radiation occurring to individuals of that type of RAPs.

1139 (132) The EIA can be used as a basis for the justification of actions aiming at the  
1140 protection of non-human species, practically of the need to further restrict discharges. The  
1141 involvement of stakeholders is recommended. The long-term preservation of the environment  
1142 is a global concern of the society, to which the application of the ethical values of radiation  
1143 protection can usefully contribute.

1144 (133) When dealing with NORM discharges in the environment, special requisites  
1145 concerning radionuclides, time interval for analysis, samples to be analysed, organisms of  
1146 concern, record keeping, and monitoring plan should be specified by considered in order to  
1147 ensure the protection of the non-human species. Long-term environmental monitoring should  
1148 be performed for regular check if the protection criteria are still met.  
1149

1150

## 5. CONCLUSIONS

1151 (134) NORM in industrial processes may be an issue from a radiological protection point  
1152 of view. The corresponding industries are diverse, they do not correspond to a sector in itself,  
1153 and they are generally big industries of economic importance. The way to address  
1154 radiological protection in industries involving NORM has been a concern for some decades.  
1155 It is a matter of justice and equity, which are ethical values of the system of radiological  
1156 protection, to consider radiological aspects as well as other industrial and chemical hazards.  
1157 Doses from industries involving NORM are variable, but they can be comparable, or greater  
1158 than, those arising from other human activities already applying the system of radiological  
1159 protection.

1160 (135) Industries involving NORM are generally licenced, although in most cases not for  
1161 radiological purposes, and these industries are used to managing risks. They should be able to  
1162 apply the criteria and requisites set for radiological protection purpose. However, experience  
1163 shows that the system of radiological protection is very specific and perceived to be difficult  
1164 to include in the management of other hazards. In such a context, the Commission  
1165 recommends a realistic and pragmatic attitude.

1166 (136) Industrial processes using NORM, although diverse, have specificities that have to  
1167 be taken into account in a protection strategy. Often, such industries have been on-going for a  
1168 long time, while the concern about radiological protection is relatively recent. They are multi-  
1169 hazards situations and in most cases the radiological risk is not dominant. While industries  
1170 involving NORM have experience in risk management, they have generally a poor awareness  
1171 of radiological protection; this can and should be developed. Industries involving NORM can  
1172 cause damages requiring remediation; however, they present no real prospect of a  
1173 radiological emergency.

1174 (137) Industries involving NORM may need to be controlled, and the system of protection,  
1175 including the principles of justification and optimisation of the protection, as well as the  
1176 corresponding dose criteria and requisites, can be applied. In order to be adapted to the  
1177 features of industries involving NORM, the Commission recommends considering as a  
1178 starting point the protection strategies already implemented by these industries to manage the  
1179 hazards they are facing and then estimating, after characterisation, the need for further action  
1180 for the protection against radiation. Such integrated approach can then be graded with a  
1181 proper balance between the different hazards, adopting a reasonable and prudent attitude and  
1182 taking into account economic and societal consideration. The involvement of the relevant  
1183 stakeholder in the decision process is also crucial.

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**REFERENCES**

- 1186 EC, 1999a. Establishment of Reference Levels for Regulatory Control of Workplaces where Materials  
1187 are Processed which Contain Enhanced Levels of Naturally Occurring Radionuclides, Directorate-  
1188 General, Environment, Nuclear Safety and Civil Protection, Radiation Protection 107.
- 1189 EC, 1999b. Radiological Protection Principles concerning the Natural Radioactivity of Building  
1190 Materials Directorate-General Environment, Nuclear Safety and Civil Protection EC, Radiation  
1191 protection 112.
- 1192 EURATOM, 2013. Council Directive 2013/59/EURATOM of 5 December 2013 laying down basic  
1193 safety standards for protection against the dangers arising from exposure to ionising radiation.
- 1194 IAEA, 1996. International Basic Safety Standards for the Protection against Ionizing Radiation and  
1195 for the Safety of Radiation Sources, Safety Series No 115, 1996, IAEA, Vienna.
- 1196 IAEA, 2005. Environmental and Source Monitoring for Purposes of Radiation Protection, Safety  
1197 Guide No.° RS-G-1.8. IAEA, Vienna
- 1198 IAEA, 2006. Assessing the Need for Radiation Protection Measures in Works Involving Minerals and  
1199 Raw Materials. Safety Reports Series No. 49, IAEA, Vienna.
- 1200 IAEA, 2010. Proceedings of Naturally Occurring Radioactive Material Symposium (NORM VI),  
1201 STI/PUB/1497, Marrakech, Morocco, March 2010. IAEA, Vienna.
- 1202 IAEA, 2015. Protection of the Public against Exposure Indoors due to Radon and Other Natural  
1203 Sources of Radiation. Specific Safety Guide No. SSG-32. IAEA. Vienna
- 1204 ICRP, 1977. Recommendations of the ICRP. ICRP Publication 26. Ann. ICRP 1(3).
- 1205 ICRP, 1983. Cost-Benefit Analysis in the Optimization of Radiation Protection. ICRP Publication 37.  
1206 Ann. ICRP 10 (2/3).
- 1207 ICRP, 1984. Principles for Limiting Exposure of the Public to Natural Sources of Radiation. ICRP  
1208 Publication 39. Ann. ICRP 14(1).
- 1209 ICRP, 1990. Optimization and Decision Making in Radiological Protection. ICRP Publication 55.  
1210 Ann. ICRP 20 (1).
- 1211 ICRP, 1991. 1990 Recommendations of the International Commission on Radiological Protection.  
1212 ICRP Publication 60. Ann. ICRP 21(1-3).
- 1213 ICRP, 1997. General Principles for the Radiation Protection of Workers. ICRP Publication 75. Ann.  
1214 ICRP 27(1).
- 1215 ICRP, 1999. Protection of the Public in Situations of Prolonged Radiation Exposure. ICRP  
1216 Publication 82. Ann. ICRP 29(1-2).
- 1217 ICRP, 2006. The Optimisation of Radiological Protection: Broadening the Process. ICRP Publication  
1218 101, Part 2. Ann. ICRP 36(3).
- 1219 ICRP, 2007a. The 2007 Recommendations of the International Commission on Radiological  
1220 Protection. ICRP Publication 103. Ann. ICRP 37(2-4).
- 1221 ICRP, 2007b. Scope of Radiological Protection Control Measures. ICRP Publication 104. Ann. ICRP  
1222 37 (5).
- 1223 ICRP, 2008. Environmental Protection - the Concept and Use of Reference Animals and Plants. ICRP  
1224 Publication 108. Ann. ICRP 38 (4-6).
- 1225 ICRP, 2010. Lung Cancer Risk from Radon and Progeny and Statement on Radon. ICRP Publication  
1226 115, Ann. ICRP 40(1).
- 1227 ICRP, 2014a. Protection of the Environment under Different Exposure Situations. ICRP Publication  
1228 124, Ann. ICRP 43(1).
- 1229 ICRP, 2014b. Radiological Protection against Radon Exposure. ICRP Publication 126. Ann. ICRP  
1230 43(3).
- 1231 ICRP, 2016. Radiological Protection from Cosmic Radiation in Aviation. ICRP Publication 132. Ann.  
1232 ICRP 45(1).
- 1233 ICRP, 2017. Occupational intakes of radionuclides: Part 3. ICRP Publication 137. Ann. ICRP  
1234 46(3/4).
- 1235 ICRP, 2018. Ethical Foundations of the System of Radiological Protection. ICRP Publication 138.  
1236 Ann. ICRP 47(1).

- 1237 Miller, H.T., Bruce, E.D. and Cook, L.M., 1991. Management of occupational and environmental  
1238 exposure to naturally occurring radioactive materials (NORM), 1991 SPE Annual Technical  
1239 Conference and EXhibition., Pt. 2, Production Operations and Engineering. Soc of Petroleum  
1240 Engineers of AIME, Richardson, TX, USA, 627-636.
- 1241 Monicard, R., Dumas, H., 1952. Radioactivité des roches sédimentaires, du pétrole brut et des eaux de  
1242 gisements. Institut Français du Pétrole 7, p. 96-102.
- 1243 Schmidt A. P., 2000, Naturally Occurring Radioactive Materials in the gas and oil industry, Origin,  
1244 transport and deposition of stable lead and  $^{210}\text{Pb}$  from Dutch gas reservoirs, Department of  
1245 Geochemistry, Utrecht University, Budapestlaan, Netherland.
- 1246 UNSCEAR, 1977, United Nations Scientific Committee on the Effects of Atomic Radiation 1977  
1247 report to the General Assembly.
- 1248 UNSCEAR, 1982. United Nations Scientific Committee on the Effects of Atomic Radiation, 1982  
1249 Report to the General Assembly, Annexe C.
- 1250 UNSCEAR, 2008. United Nations Scientific Committee on the Effects of Atomic Radiation 2008  
1251 report to the General Assembly, Annexe B.
- 1252 UNSCEAR, 2016. United Nations Scientific Committee on the Effects of Atomic Radiation 2016  
1253 report to the General Assembly, Annexe B.
- 1254

1255

**ANNEX A. ACTIVITIES GIVING RISE TO NORM EXPOSURES**

1256 (A 1) The main activities giving rise to NORM exposure are the following.

1257 (A 2) **Extraction of rare earth elements.** The most important source of rare earth  
1258 elements are monazite (Ce, La, Nd, Th)PO<sub>4</sub> and bastnaesite. The crystal structure of monazite  
1259 accepts uranium and thorium and is the most common radioactive mineral on Earth. Activity  
1260 concentration ranges from 5,000 – 350,000 Bq kg<sup>-1</sup> of <sup>232</sup>Th, and 10,000 – 50,000 Bq kg<sup>-1</sup> of  
1261 <sup>238</sup>U (UNSCEAR, 2008). During the extraction process to obtain rare earth elements (by  
1262 mechanical or chemical means), inhalation of dust and external gamma radiation to workers  
1263 may occur. Furthermore, effluents, residues and waste from the extraction process contain  
1264 thorium, radium and uranium at concentration higher than in the feedstock (EC, 1999a).  
1265 Waste in the form of mill tailing can be used for landfill material or may need specific  
1266 management.

1267 (A 3) **Production and use of metallic thorium and its compounds.** Thorium under an  
1268 oxide form occurs in many minerals, notably monazite. It can be extracted by concentrating  
1269 minerals and decomposing them with acid to obtain thorium salts; which is the raw material  
1270 for the production thorium under metallic form. Thorium is used in a number of materials,  
1271 usually as an additive (e.g. thoriated tungsten isolated welding electrodes, that usually contain  
1272 100,000 Bq kg<sup>-1</sup> of <sup>232</sup>Th and <sup>228</sup>Th (EC, 1999a)) or alloy (e.g. magnesium thorium used in jet  
1273 engines; activity about 70,000 Bq kg<sup>-1</sup>) and as thorium nitrate in the manufacture of gas  
1274 mantles. Small quantities of thorium can be found in many products: glass, airport runaway  
1275 lights, lamp starters etc. Producing material containing thorium can give rise to external  
1276 gamma exposure and internal exposure through the inhalation of dust. The process also  
1277 generates solid wastes and effluents that may need to be monitored and controlled.

1278 (A 4) **Mining and processing of ores (other than uranium).** According to International  
1279 Labour Organization, mining is an extensive industry that account for about 1 % of the world  
1280 workforce (that is to say about 30 million workers, including some 12 million in the coal  
1281 mining). The main source of exposure in mining operation is radon, however, exposure due  
1282 to long-term radionuclides through gamma external exposure and the inhalation and ingestion  
1283 of mineral dusts can be important in certain situations.

1284 (A 5) The processing of ores may be also concerned by the use of NORM and the  
1285 exposure situations for workers differ considerably with respect to the type of industry, the  
1286 conditions at workplaces, the radionuclides involved and their physical and chemical forms  
1287 etc. The natural radionuclides involved in extractive industries end up in the products and/or  
1288 in the effluents and/or wastes. Sediment discharges in waste water into the environment have  
1289 been measured with activity up to 55,000 Bq kg<sup>-1</sup> of <sup>226</sup>Ra and 15,000 Bq kg<sup>-1</sup> of <sup>228</sup>Ra  
1290 (IAEA, 2003).

1291 (A 6) **Extraction of oil and gas.** The water contained in oil and gas geological formations  
1292 contains <sup>228</sup>Ra, <sup>226</sup>Ra and <sup>224</sup>Ra dissolved from the reservoir rock, together with their decay  
1293 progenies. When this water is brought to the surface with the oil and gas, changes in  
1294 temperature and pressure can lead to the precipitation of radium rich sulphate and calcium  
1295 carbonate scales on the inner walls of production equipment (pipes, valves, pumps etc.).  
1296 Depending on the age of the scale, significant amount of <sup>210</sup>Pb and <sup>228</sup>Th may grow in with  
1297 their respective radioactive parents (IAEA, 2006). In any case, the activity concentrations in  
1298 scale are difficult to predict and activity concentration has been reported as being less than  
1299 1,000 to around 1,000,000 Bq kg<sup>-1</sup> of <sup>226</sup>Ra (EC, 1999a). The radium isotopes and their  
1300 progeny can also appear in sludges in separators and skimmer tanks (more details can be  
1301 found in Table 5 of IAEA (2003)). The main radiological protection issue associated with the  
1302 scale are external gamma exposure of workers, especially where scales are deposited and

1303 internal exposure by staff removing the scale during maintenance and decommissioning.  
 1304 Figures related to activity concentration in oil, gas, scale and sludge are given in Table A.1  
 1305 (IAEA, 2003, 2011).

1306 (A 7) Operators may try to prevent deposition of scales through the application of  
 1307 chemical scale inhibitors in the water. As a result, the radium isotopes will pass through the  
 1308 production system and be released with the produced water. In the same way, the new  
 1309 technique of ‘fracking’ (hydraulic fracturing) for gas production also releases NORM in drill  
 1310 cuttings and water. For example, US Geological Survey shows median activity concentration  
 1311 for produced water of 200 Bq L<sup>-1</sup> (USGS, 2011).

1312

1313

Table A.1. Range of concentrations of radionuclides in oil, gas and by-products.

	Crude oil (Bq kg <sup>-1</sup> )	Natural gas (Bq m <sup>-3</sup> )	Produced water (Bq L <sup>-1</sup> )	Hard scale (Bq kg <sup>-1</sup> )	Sludge (Bq kg <sup>-1</sup> )
<sup>238</sup> U	0.0001 – 10		0.0003 – 0.1	1 – 500	5 – 10
<sup>226</sup> Ra	0.1 – 40		0.002 – 1200	100 – 15,000,000	5 – 800,000
<sup>210</sup> Po	0 – 10	0.002 – 0.08		20 – 1500	4 – 160,000
<sup>210</sup> Pb		0.005 – 0.02	0.05 – 190	20 – 75,000	100 – 1,300,000
<sup>222</sup> Rn	3 – 17	5 – 200,000			
<sup>232</sup> Th	0.3 – 2		0.0003 – 0.001	1 – 2	2 – 10
<sup>228</sup> Ra	3 – 17		0.3 – 180	50 – 2,800,000	500 – 50,000
<sup>224</sup> Ra			0.5 – 40		

1314

1315 (A 8) **Manufacture of titanium dioxide.** Titanium can be extracted from ilmenite  
 1316 (which contain monazite as impurity) and rutile which may contain elevated levels of both  
 1317 <sup>232</sup>Th and <sup>238</sup>U. The radiological exposure from titanium dioxide production varies with the  
 1318 type and source of ore and the process. Ore concentration activity of <sup>238</sup>U and <sup>232</sup>Th ranges  
 1319 from 7 to 9,000 Bq kg<sup>-1</sup> (EC, 1999a). The separation process could give rise to radiological  
 1320 hazards from dust inhalation and external gamma radiation emanating from large stockpiles  
 1321 of material. Precipitate containing isotopes of radium may occur during the process and be  
 1322 found in the waste (at activity concentration up to 1,600,000 Bq kg<sup>-1</sup> (IAEA, 2006)).

1323 (A 9) **The phosphate processing industry.** Phosphate rock is the starting material for the  
 1324 production of all phosphate products and is the main source of phosphorous for fertilisers.  
 1325 The radionuclides content of the ore varies greatly depending of its origin (IAEA, 2003) and  
 1326 is generally less than 3,000 Bq kg<sup>-1</sup> of uranium. The phosphate processing can be divided into  
 1327 the mining and milling of phosphate ore – there is no significant enhancement of activity  
 1328 concentration during this phase, but exposure through inhalation and external exposure may  
 1329 occur – and the manufacturing of phosphate products by wet or thermal process.

1330 (A 10) Most phosphate rock is treated with sulphuric acid to produce phosphoric acid (wet  
 1331 process). The phosphoric acid can be combined with ammonia to make ammonium phosphate  
 1332 which is the basis of mixed fertiliser. The production of phosphoric acid generates large  
 1333 quantities of phosphogypsum – evidence suggests that radium isotopes are more readily  
 1334 retained in the phosphogypsum (EC, 1999a). Phosphogypsum is also used as building  
 1335 material and in agriculture. Environmental protection issues (regarding radiological impact  
 1336 and toxicity) may arise from the disposal of phosphogypsum in stockpile or by discharge into  
 1337 surface water bodies.

1338 (A 11) Furthermore, radium scales and sediments can be formed inside equipment during  
1339 the wet process, and the radium activity concentrations in the scales vary from values similar  
1340 to those in the original ore up to 1,000 times greater (IAEA, 2006), leading to possible  
1341 exposure by external gamma radiation and/or inhalation of dust during maintenance and  
1342 decommissioning.

1343 (A 12) In the thermal process, phosphate is crushed and mixed with silica and coke to be  
1344 burnt in furnace at 1500° C. At this temperature, phosphorus vapour is produced and can be  
1345 condensed and removed as liquid or solid. The elemental phosphorus can be used for the  
1346 production of high purity phosphoric acid and other phosphorus products. During this  
1347 process, volatile radionuclides like  $^{210}\text{Pb}$  and  $^{210}\text{Po}$  are produced as well and become  
1348 concentrated in the precipitator (typical concentration are 50,000 to 500,000 Bq kg<sup>-1</sup> (EC,  
1349 1999a)) while thorium and uranium are retained in the slag (activity concentration ranges  
1350 between 1 and 3000 Bq kg<sup>-1</sup>). Dust and slag may present NORM exposure to workers and to  
1351 public when used as construction material in cement.

1352 (A 13) **The zircon and zirconia industries.** Zircon (or zirconium silicate) is recovered  
1353 from beach sands. The sand is pre-processed in very large quantities by gravimetric and  
1354 electromagnetic sorting to separate the mineral sands. Exposure from NORM to workers may  
1355 arise due to the inhalation of dust and external irradiation from the large amount of material.  
1356 When chemical processing of zircon is used, effluents may contain NORM. A very large  
1357 range of activity concentrations are reported for zirconium silicate, from 200 – 74,000 Bq kg<sup>-1</sup>  
1358 of  $^{238}\text{U}$  and 400 – 40,000 Bq kg<sup>-1</sup> of  $^{232}\text{Th}$  (EC, 1999a; IAEA, 2012). Most zircon sand is  
1359 used as opacifier in fine ceramics, enamels, glazes and sanitary ware. Zircon sands can also  
1360 be manufactured as refractory component by mixing the sand with alumina and sodium  
1361 carbonate and smelting the mixture.  $^{210}\text{Pb}$  and  $^{210}\text{Po}$  are volatilised and end up in the fume  
1362 collection system (up to 200,000 Bq kg<sup>-1</sup> of  $^{210}\text{Pb}$  and 600,000 Bq kg<sup>-1</sup> of  $^{210}\text{Po}$  (IAEA,  
1363 2006)).

1364 (A 14) **Production of metal.** Largely depending on the origin of metal ore, the extraction  
1365 of many metals may give rise to exposure to NORM because smelting and refining at high  
1366 temperatures may volatilise  $^{210}\text{Pb}$  and  $^{210}\text{Po}$  from ore that can lead to exposure by inhalation  
1367 during the process and later when these radionuclides have been precipitated and  
1368 concentrated (up to 200,000 Bq kg<sup>-1</sup> (IAEA, 2006, 2013)). Non-volatile radionuclides may be  
1369 concentrated in the slag (concentration range from less than 1000 to more than 10,000 Bq kg<sup>-1</sup>  
1370 <sup>1</sup>). Such exposures could be found in the production of tin, copper, iron, steel, aluminium,  
1371 niobium/tantalum, bismuth, etc.

1372 (A 15) **Extraction and combustion of coal.** Most fossil fuels and notably coal contain  
1373 uranium and thorium and their decay products, as well as  $^{40}\text{K}$ . The activity concentrations are  
1374 generally not elevated and depend on the region of origin and its geology (examples of  
1375 figures are given in p. 184 of UNSCEAR (2016)). However, UNSCEAR 2016 estimated that  
1376 occupational exposure due to coal mining was 23,000 man.Sv for the 2002-2003 period and  
1377 that annual average effective dose for Chinese coal miners (90% of the workforce) was 2.75  
1378 mSv per year. Due to the amount of material, the quantities of radionuclides involved are  
1379 noteworthy. For example, over 8,000 millions of tons of coals were extracted in 2014  
1380 (according to British Petroleum Statistical Review of World Energy) and by considering the  
1381 lower values of 4 ppm of uranium and 10 ppm of thorium, 32,000 tons of uranium and  
1382 80,000 tons of thorium can be considered as being extracted as well.

1383 (A 16) The combustion of coal fuel to produce heat and electricity will generate fly ash  
1384 and the heavier bottom ash or slag. The concentration of radionuclides in the bottom ash and  
1385 slag tends to be higher than in the coal (around 10 times), but generally do not exceed 5,000  
1386 Bq kg<sup>-1</sup> (IAEA, 2006) – range of radionuclides activities in ashes are presented in Table A.2

1387 (UNSCEAR, 1982). The volatile materials such as lead and polonium can be released to the  
 1388 atmosphere or, in modern power stations, retained and can accumulate in fly ash as well as  
 1389 the inner surface of the burner ( $^{210}\text{Po}$  activity concentration above  $100,000 \text{ Bq kg}^{-1}$  in the  
 1390 deposited scale have been reported). Gas desulphurisation results in additional sludge and  
 1391 gypsum. The use of coal combustion residues (ash, gypsum) in cement or concrete is a  
 1392 worldwide practice.

1393

1394 Table A.2. Ranges of radionuclides activities in coal ash and slag.

	Potassium ( $\text{Bq kg}^{-1}$ )	Thorium series ( $\text{Bq kg}^{-1}$ )	Uranium series ( $\text{Bq kg}^{-1}$ )
Bottom ash (slag)	240 – 1200	44 – 560	48 – 3900
Fly ash (collected)	260 – 1500	30 – 300	30 – 2000
Fly ash (escaping)	260	100 – 160	20 – 5500

1395

1396 (A 17) **Water treatment.** Treatment of underground water is a common practice to  
 1397 remove salts and other contaminants. Various processes may be used; such as filters or ion  
 1398 exchange resins. Radionuclides of natural origin present in the water may accumulate in  
 1399 water treatment wastes (filter sludge). The activity concentration in such waste is generally  
 1400 moderated but can reach  $10,000 \text{ Bq kg}^{-1}$  (IAEA, 2006).

1401 (A 18) **Building materials.** The use of some building materials may lead to elevated  
 1402 indoor radiation levels when they contain elevated levels of radionuclides including  
 1403 particularly  $^{226}\text{Ra}$ ,  $^{232}\text{Th}$  and  $^{40}\text{K}$ . The building material may be of natural origin or contain  
 1404 materials derived from industrial processes such as those listed above. Values for activity  
 1405 concentration in  $\text{Bq kg}^{-1}$  in some building materials are given in Table A.3 (UNSCEAR,  
 1406 1982; IAEA, 2003).

1407 (A 19) Activity concentration guidelines for the use of NORM in building material have  
 1408 been developed in Europe through the use of an Activity Concentration Index, ACI,  
 1409 considering  $^{226}\text{Ra}$ ,  $^{232}\text{Th}$  and  $^{40}\text{K}$  activity in the material (EC, 1999b; EURATOM, 2013).

1410

1411 Table A.3. Examples of activity concentration in  $\text{Bq kg}^{-1}$  for some building materials.

Material	$^{226}\text{Ra}$	$^{232}\text{Th}$	$^{40}\text{K}$
Concrete	1 – 250	1 – 190	5 – 1570
Aerated concrete	11,000	1 – 220	180 – 1,600
Clay bricks	1 – 200	1 – 200	60 – 2,000
Sand-lime bricks and sandstone	18,000	11,000	5 – 700
Natural gypsum	<1 – 70	<1 – 100	7 – 280
Granite	100	80	1,200
Lithoid tuff	130	120	1,500
Pumice stone	130	130	1,100
Cement	7 – 180	7 – 240	24 – 850
Tiles	30 – 200	20 – 200	160 – 1,410
Phosphogypsum	4 – 700	19,000	25 – 120
Blast furnace slag stone and cement	30 – 120	30 – 220	-

1412

1413 (A 20) **Legacy sites.** There are also several sites with residues from former installations  
 1414 around the world. Most of these sites are contaminated with natural radionuclides from  
 1415 former industries involving NORM. In some cases, these sites have been identified and  
 1416 successfully remediated. However, it is almost certain that a significant number of  
 1417 contaminated sites from former industries involving NORM have yet to even be identified.

1418 (A 21) From the above paragraphs, industries involving NORM process a wide range of  
1419 raw materials with large variation of activity concentrations, producing a variety of products,  
1420 by-products and wastes, which also have an even larger variation in activity concentrations.  
1421 These industries may or may not be of concern depending on the activity concentrations in  
1422 the raw materials handled, the processes adopted, the uses to which final products are put, the  
1423 re-use and recycling of residues and the disposal of wastes.

## 1424 **A.1. References**

- 1425 EC, 1999a. Establishment of Reference Levels for Regulatory Control of Workplaces where Materials  
1426 are Processed which Contain Enhanced Levels of Naturally Occurring Radionuclides, Directorate-  
1427 General, Environment, Nuclear Safety and Civil Protection, Radiation Protection 107.
- 1428 EC, 1999b, Radiological Protection Principles concerning the Natural Radioactivity of Building  
1429 Materials Directorate-General Environment, Nuclear Safety and Civil Protection EC, Radiation  
1430 protection 112.
- 1431 EURATOM, 2013. Council Directive 2013/59/EURATOM of 5 December 2013 laying down basic  
1432 safety standards for protection against the dangers arising from exposure to ionising radiation.
- 1433 IAEA, 2003. Extent of Environmental Contamination by Naturally Occurring Radioactive Material  
1434 (NORM) and Technological Options for Mitigation, Technical Reports Series No. 419, IAEA,  
1435 Vienna.
- 1436 IAEA, 2006. Assessing the Need for Radiation Protection Measures in Works Involving Minerals and  
1437 Raw Materials. Safety Reports Series No. 49, IAEA, Vienna.
- 1438 IAEA, 2011. Radiation Protection and NORM Residue Management in the Production of Rare Earths  
1439 from Thorium Containing Minerals, Safety Reports Series No. 68, IAEA, Vienna.
- 1440 IAEA, 2012. Radiation Protection and NORM Residue Management in the Titanium Dioxide and  
1441 Related Industries, Safety Reports Series No. 76, IAEA, Vienna.
- 1442 IAEA, 2013. Radiation Protection and Management of NORM Residues in the Phosphate Industry,  
1443 Safety Reports Series No. 78, IAEA, Vienna.
- 1444 UNSCEAR, 1982. United Nations Scientific Committee on the Effects of Atomic Radiation, 1982  
1445 Report to the General Assembly, Annexe C.
- 1446 UNSCEAR, 2008. United Nations Scientific Committee on the Effects of Atomic Radiation 2008  
1447 report to the General Assembly, Annexe B.
- 1448 UNSCEAR, 2016. United Nations Scientific Committee on the Effects of Atomic Radiation 2016  
1449 report to the General Assembly, Annexe B.
- 1450 USGS, 2011. Rowan, E.L., Engle, M.A., Kirby, C.S., and Kraemer, T.F., Radium Content of Oil and  
1451 Gas-Field Produced Waters in the Northern Appalachian Basin (USA), Summary and Discussion  
1452 of data, U.S. Geological Survey Scientific Investigations Report 2011–5135, 31 p. (available  
1453 online at <http://pubs.usgs.gov/sir/2011/5135/>).
- 1454

1455

## GLOSSARY

1456 Adventitious

1457 Happening as a result of an external factor or chance rather than design or inherent  
1458 nature. In this report, the word is used in a sense close to inadvertent, coincidental,  
1459 unintentional, unintended.

1460 Categories of exposure

1461 The Commission distinguishes between three categories of radiation exposure for  
1462 humans: occupational, public, and medical, and also considers environmental  
1463 exposure for flora and fauna. Distinction made between human and non-human biota  
1464 takes into account the context in which they are exposed.

1465 Contamination

1466 The presence of unwanted levels of radioactive material on or in structures, areas,  
1467 objects, biota and people.

1468 Discharge

1469 Controlled release of (usually gaseous or liquid) radioactive material to the  
1470 environment.

1471 Dose criteria

1472 Quantitative values for the practical implementation of the radiological protection  
1473 system, expressed in terms of dose or derived quantities.

1474 Effluent

1475 Fluid - treated or untreated - that flows out of a treatment plant, sewer, or industrial  
1476 outfall.

1477 Emergency exposure situations

1478 An exposure situation resulting from a loss of control of a source, or from intentional  
1479 misuse of a source, which requires urgent and timely actions in order to avoid or  
1480 mitigate exposure.

1481 Employer

1482 An organisation, corporation, partnership, firm, association, trust, estate, public or  
1483 private institution, group, political or administrative entity, or other persons  
1484 designated in accordance with national legislation, with recognized responsibility,  
1485 commitment, and duties towards a worker in her or his employment by virtue of a  
1486 mutually agreed relationship. A self-employed person is regarded as being both an  
1487 employer and a worker.

1488 Environmental exposure

1489 Radiation exposure of biota in the natural environment resulting from human  
1490 activities.

1491 Environmental reference level

1492 This term refers to the Derived Consideration Reference Level (DCRL) introduced in  
1493 *Publication 108*, which is a band of dose rate within which there is likely to be some  
1494 chance of deleterious effects of ionising radiation occurring to individuals of that type  
1495 of reference animal or plant (derived from a knowledge of defined expected  
1496 biological effects for that type of organism) that, when considered together with other  
1497 relevant information, can be used as a point of reference to optimise the level of effort  
1498 expended on environmental protection, dependent upon the overall management  
1499 objectives and the relevant exposure situation.

1500 Existing exposure situations

1501 An exposure situation resulting from a source that already exists, with no intention to  
1502 use the source for its radioactive properties, before a decision to control the resulting  
1503 exposure is taken. Decisions on the need to control the exposure may be necessary  
1504 but not urgent.

1505 Exposure pathway

1506 A route by which radiation or radionuclides can reach humans and non-human biota  
1507 and cause exposure.

1508 Graded approach

1509 The scheme recommended for implementing the system of radiological protection in  
1510 a way that is proportionate to the magnitude and likelihood of the risk, and the  
1511 complexity of the exposure situation and the prevailing circumstances.

1512 Medical exposure

1513 Exposure incurred by patients as part of their own medical or dental diagnosis or  
1514 treatment, by persons, other than those occupationally exposed, knowingly, while  
1515 voluntarily helping in the support and comfort of patients; and by volunteers in a  
1516 programme of biomedical research involving their exposure.

1517 Member of the public

1518 Any individual who is subject to a public exposure.

1519 NORM (naturally occurring radioactive material)

1520 Material containing no significant amounts of radionuclides other than naturally  
1521 occurring radionuclides, in which the activity concentrations of the naturally  
1522 occurring radionuclides have been changed by some process and giving rise to  
1523 enhanced exposure to human and non-human species.

1524 Occupational exposure

1525 Exposure incurred by individuals as a result of their work in circumstances for which  
1526 the exposure can be reasonably considered as deserving to be managed individually.  
1527 This has to be evaluated on a case by case basis. There is no single answer that is  
1528 always applicable. It is a value judgement. Factors to be considered include the level  
1529 of exposure, the potential for unforeseen circumstances or large exposures because of  
1530 the characteristics of the source.

1531 Operating management

1532 The person or group of persons that directs, controls, and assesses an organization at  
1533 the highest level. Many different terms are used, including chief executive officer,  
1534 director general, managing director, and executive group.

1535 **Planned exposure situations**

1536 An exposure situation resulting from the deliberate introduction and operation of  
1537 radiation sources, used for their radioactive properties. For this type of situation, the  
1538 use of the source is understood, and as such the exposures can be anticipated and  
1539 controlled from the beginning.

1540 **Principle of justification**

1541 Decisions that alter (i.e. introduce, reduce or remove) the radiation exposure situation  
1542 should, overall, do more good than harm. This means that, by introducing a new  
1543 radiation source, or by overall reducing existing or emergency exposures, one should  
1544 achieve sufficient individual or societal benefit to offset any harm including radiation  
1545 detriment to humans and the environment.

1546 **Principle of optimisation**

1547 The likelihood of incurring exposures, and the magnitude of their individual doses,  
1548 should be kept as low as reasonably achievable, taking into account societal,  
1549 economic and environmental factors. In order to avoid inequities in the dose  
1550 distribution, there must be consideration of the number of people exposed and  
1551 restrictions on individual doses.

1552 **Protection strategy**

1553 The set of combined protective actions implemented, for a given exposure situation  
1554 and prevailing circumstance, to keep or reduce exposure as low as reasonably  
1555 achievable.

1556 **Protective action**

1557 Action taken in an exposure situation to reduce or prevent exposure. The action can  
1558 be taken at the source, at points in the exposure pathway, or occasionally by  
1559 modifying the location, habits or working conditions of the exposed individuals.

1560 **Public exposure**

1561 Exposure incurred by individuals from radiation sources, other than occupational and  
1562 medical exposure.

1563 **Reference animal or plant**

1564 A hypothetical entity, with the assumed basic biological characteristics of a particular  
1565 type of animal or plant, as described to the generality of the taxonomic level of  
1566 Family, with defined anatomical, physiological, and life-history properties, that can  
1567 be used for the purposes of relating exposure to dose, and dose to effects, for that type  
1568 of living organism.

1569 **Reference level**

1570 The value of dose used to drive the optimisation process in existing and emergency  
1571 exposure situations. The value of a reference level will be selected within the bands

1572 recommended by the Commission according to the prevailing circumstances. This  
1573 selection should consider the actual individual dose distribution, with the objective of  
1574 identifying those exposures that warrant specific attention and should be reduced as  
1575 low as reasonably achievable.

1576 **Representative organism (non-human biota)**

1577 An organism or group of organisms receiving a dose that is representative of the  
1578 doses to the most exposed individuals in an exposed group from a given source,  
1579 excluding extreme habits.

1580 **Residue**

1581 Radioactive materials that have remained in the environment from early operations  
1582 and from accidents. Residue from one industry may be used as feedstock in another  
1583 industry, and as such are not classified as waste.

1584 **Stakeholder**

1585 A stakeholder is a person, a group or organisation with an interest or concern in an  
1586 issue.

1587 **Waste**

1588 Any radioactive material that will be or has been discarded, being of no further use.

1589 **Worker**

1590 Any person who is employed or self-employed, whether full time, part time or  
1591 temporarily, by an employer, and who has recognised rights and duties in relation to  
1592 her/his job.  
1593  
1594

1595

**ACKNOWLEDGEMENTS**

1596 At its meeting in Berlin (Germany) in October 2007, the Main Commission of the  
1597 International Commission on Radiological Protection (ICRP) approved the formation of a  
1598 Task Group 76, reporting to Committee 4, to develop guidance on radiological protection  
1599 against exposure from naturally occurring radioactive material (NORM).

1600

1601 The terms of reference of the Task Group were to develop a report on the application of the  
1602 2007 Commission's recommendations (ICRP, 2007a) on radiological protection of workers,  
1603 the public and environment to exposures resulting from industrial processes with NORM.  
1604 The aim of the Task Group was to develop recommendations to cover the broad range of  
1605 activities associated with the processing, manufacturing or use and disposal of materials with  
1606 enhanced levels of naturally occurring radionuclides. The report should also clarify the issues  
1607 concerning the type of exposure situation, the categories of exposure, and the basic principles  
1608 to be applied for the management of NORM.

1609

1610 The initial membership of Task Group 76 was as follows:

1611

P. Burn, Chair	G. Loriot	L. Setlow
A. Canoba	M. Markkanen	
A. Liland	S. Romanov	

1612

1613 The corresponding members were:

1614

Å. Wiklund	D. Wymer
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1616

1617 A new membership was designated in 2013, as follow:

1618

J-F. Lecomte, Chair	A. Liland	P. Shaw (2013-2017)
D. da Costa Lauria	F. Liu	
P. Egidi	M. Markkanen	

1619

1620 Corresponding members were:

1621

P.P. Haridasan (2013-2015)	H.B. Okyar (2015-2017)	S. Mundigl
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1623

1624 Committee 4 critical reviewers were:

1625

A. Canoba	T. Pather (2013-2017)	Gillian Hirth (2017-2021)
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1627

1628 Main Commission critical reviewers were:

1629

C-M. Larsson	S. Romanov
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1631

1632 In addition, Sylvain Andresz (CEPN) acted as secretary of the Task Group and provided  
1633 fruitful scientific assistance. A helpful contribution was also received from Luiz Matta,  
1634 Jelena Popic, Bo Wang, a French mirror group, and through the ICRP consulting process.

1635

1636 The membership of the Main Commission at the time of approval of this publication was:

1637

1638 Chair: C. Cousins, *UK*1639 Vice-Chair: J. Lochard, *France*1640 Scientific Secretary: C.H. Clement, *Canada*; [sci.sec@icrp.org](mailto:sci.sec@icrp.org)

1641

1642 K.E. Applegate, *USA*S. Liu, *China***Emeritus Members**1643 S. Bouffler, *UK*S. Romanov, *Russia*R.H. Clarke, *UK*1644 K.W. Cho, *Korea*W. Rühm, *Germany*F.A. Mettler Jr., *USA*1645 D.A. Cool, *USA*R.J. Pentreath, *UK*1646 J.D. Harrison, *UK*R.J. Preston, *USA*1647 M. Kai, *Japan*C. Streffer, *Germany*1648 C.-M. Larsson, *Australia*E. Vaño, *Spain*1649 D. Laurier, *France*

1650

1651 The membership of Committee 4 during the period of preparation of this report was:

1652

1653 (2009-2013)

J. Lochard (Chair)

M. Kai

A. McGarry

W. Weiss (Vice-Chair)

J-F. Lecomte (Secretary)

K. Mrabit

P. Burns

H. Liu

S. Shinkarev

P. Carboneras

S. Liu

J. Simmonds

D.A. Cool

S. Magnusson

A. Tsela

T. Homma

G. Massera

W. Zeller

1654

1655 (2013-2017)

D.A. Cool (Chair)

E. Gallego

A. Nisbet

K-W. Cho (Vice-Chair)

T. Homma

D. Oughton

F. Bochud

M. Kai

T. Pather

M. Boyd

J-F. Lecomte (Secretary)

S. Shinkarev

A. Canoba

S. Liu

J. Takala

M. Doruff

A. McGarry

1656

1657 (2017-2021)

D.A. Cool (Chair)

D. Copplestone

Y. Mao

K.A. Higley (Vice-Chair)

E. Gallego

N. Martinez

N. Ban

G. Hirth

A. Nisbet

F. Bochud

T. Homma

T. Schneider

M. Boyd

C. Koch

S. Shinkarev

A. Canoba

J-F. Lecomte (Secretary)

J. Takala

1658